#### March 28, 2008

MEMORANDUM TO: Marc L. Dapas, Deputy Regional Administrator, RI

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FROM: Frederick D. Brown, Director /RA/

Division of Inspection and Regional Support

Office of Nuclear Reactor Regulation

SUBJECT: REACTOR OVERSIGHT PROCESS ANNUAL SELF-ASSESSMENT

PERFORMANCE METRICS

The Reactor Oversight Process (ROP) self-assessment process uses objective measures and predetermined criteria to monitor the performance of the ROP as described in Inspection Manual Chapter (IMC) 0307, "Reactor Oversight Process Self-Assessment Program." The calendar year (CY) 2007 ROP self-assessment was performed in accordance with IMC 0307 which was recently revised to consolidate and clarify several of the performance metrics. These metrics rely on information from various sources, including the Reactor Program System, the inspection program, periodic independent audits, stakeholder surveys, and public comments. The staff collects data quarterly and uses pre-established success criteria to analyze the data.

The Nuclear Regulatory Commission (NRC) solicited comments on the eighth year of ROP implementation from external stakeholders in a *Federal Register* notice in October 2007. Of the seven respondents, four were from the utilities and/or their representatives, one was from a State agency, and two were from public interest groups or public citizens. There was no internal survey conducted in CY 2007 consistent with its biennial frequency as defined by IMC 0307. The next survey to internal stakeholders will be conducted during the fourth quarter of CY 2008.

Based on the NRC staff's review, most of the metrics met the established criteria. Two metrics in the assessment (AS) area, two metrics in the performance indicator (PI) area, one metric in the inspection (IP) area and one metric in the significance determination process (SDP) area did not meet the established criteria. All Overall (O) ROP metrics met the established criteria. The NRC staff's corrective actions to address these issues are discussed in the following paragraphs, in the enclosed metric analyses, and in the CY 2007 ROP self-assessment Commission paper.

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#### PI Program Results

Two of the eight PI metrics did not meet the established criteria. PI-3, "Timely Indication of Declining Safety Performance," which tracks PIs that cross multiple thresholds, did not meet established criteria because there were four occurrences at 3 sites, an increase from last year. PI-4, "PI Program Provides Insights to Help Ensure Plant Safety," did not meet its criteria because public and state respondents gave feedback that the PIs do not provide an adequate indication of declining safety performance. The NRC staff recognizes the need to improve the PI Program and is reviewing the program to provide more timely and meaningful indications of declining plant performance to ensure plant safety.

#### Inspection Program Results

One of the nine inspection program metrics did not meet established criteria. IP-5, "Temporary Instructions (TI) Are Completed Timely," failed to meet the established criteria of completing all TIs within the TI requirements. Only one site did not meet the criteria. The NRC staff is considering revising the metric criteria from "all" to 95 percent, consistent with other metrics, in the next revision of IMC 0307.

#### SDP Results

One of the seven SDP metrics, SDP-4, "The SDP Provides an Objective and Understandable Regulatory Response to Performance Issues," did not meet its criteria. This is based on a stable negative perception by external stakeholders over the past seven years of ROP implementation. Findings are continuously under review by the NRC to determine the need for adjustments to the SDPs in this area and several recent changes have been made. SDP-3, "Inspection Staff Is Proficient and Find Value in Using the SDP," was not analyzed in the CY 2007 metric report because the internal survey was not conducted in CY 2007, consistent with its biennial frequency as defined by IMC 0307.

#### Assessment Program Results

Two of the eight assessment program metrics did not meet established criteria. AS-4, "The NRC's Response to Performance Issues Is Timely," counts the elapsed time between issuance of an assessment letter and the respective supplemental inspection exit meeting date. The average elapsed time increased from last year. AS-7, "Degradations in Plant Performance are Gradual and Allow Adequate Agency Engagement of the Licensees," tracks the number of instances plants moved more than one column to the right in the Action Matrix. There were five occurrences at 4 sites, an increase from last year. The NRC staff plans to address these issues during CY 2008 and continue to monitor them closely. AS-8, "Perceived Effectiveness of Safety Culture Enhancements to ROP," was not applicable this assessment period. A more meaningful assessment of the AS-8 metric criterion will be achieved in follow-on years as the safety culture changes have been implemented for a longer period.

#### Overall ROP Results

All 16 metrics in the Overall ROP area met the established criteria.

#### Conclusions and Next Steps

The performance metrics provide the staff with valuable insights and lessons learned that lead to continued improvements in ROP effectiveness. This report provides a significant input into the annual ROP self-assessment and the resulting Commission paper. Aspects of this report, particularly the six metrics not met, will be discussed in the self-assessment paper under the respective program areas. The NRC staff will prepare and distribute a consolidated response to stakeholder comments from the CY 2007 external survey.

Enclosure: As stated

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Enclosure: As stated

### DISTRIBUTION: DRP/DRS

ADAMS ACCESSION NUMBER: ML080350368

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### **ENCLOSURE**

### Calendar Year 2007 Analysis of the

**Reactor Oversight Process Self-Assessment Metrics** 

(In Accordance with IMC 0307, Appendix A)

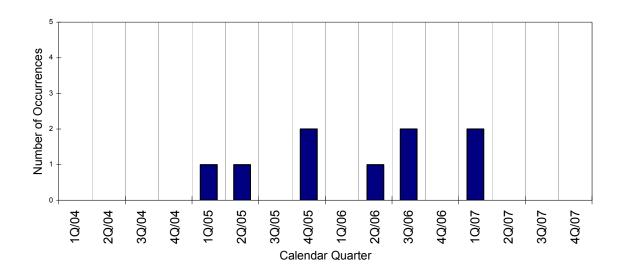
#### PI-1 Consistent Results Given Same Guidance

**Definition:** 

Independently verify PIs using Inspection Procedure (IP) 71151, "PI Verification." Count all performance indicators (PI) that either (a) result in a crossed threshold based on a data correction by the licensee (as noted in the resultant inspection report), or (b) have been determined to be discrepant by the staff in accordance with IP 71150, "Discrepant or Unreported Performance Indicator Data."

**Criteria:** Expect few occurrences, with a stable or declining trend.

Goals Supported: Objective, Predictable, Ensure Safety



Comments:

The graph represents the number of significant deficiencies or discrepant PIs reported for each quarter. Significant discrepancies are issues identified by the NRC during a PI verification inspection that caused the PI to cross a threshold.

Analysis:

During this assessment period there were two PIs that resulted in a crossed threshold based on a data correction by the licensee and no PIs were identified as discrepant.

Ginna crossed the threshold from Green to White for the Emergency Response Organization (ERO) Drill Participation PI when the data for 4Q06 was re-reported with the 1Q07 data submittal. Ginna moved to Yellow for 1Q07 as a result of the changed data.

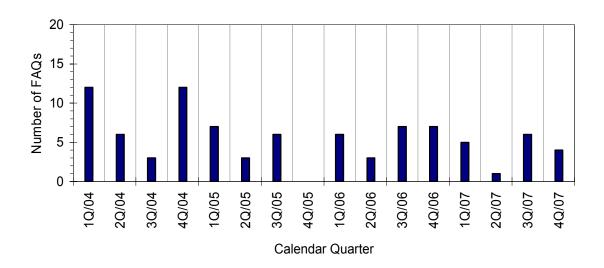
Vogtle Unit 2 crossed the threshold from Green to White for the Mitigating System Performance Index - Cooling Water Systems PI when the data for 4Q06 was re-reported with the 1Q07 data submittal.

PI-2 Questions Regarding Interpretation of PI Guidance

**Definition:** Quarterly, count the number of frequently asked questions (FAQs).

**Criteria:** Expect low numbers, with a stable or declining trend.

**Goals Supported:** Understandable, Risk-Informed, Predictable



**Comments:** Each quarter represents the total number of new FAQs introduced during the

ROP NRC/Industry Working Group meetings held during the respective quarter.

**Analysis:** There is currently a stable long term trend. The number of FAQs introduced in

CY 2007 has been comparable to previous years.

#### PI-3 Timely Indication of Declining Safety Performance

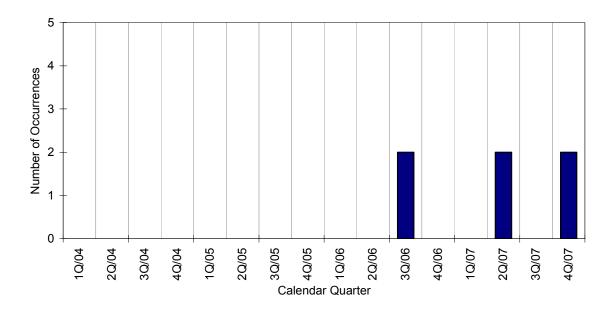
**Definition:** Quarterly, track PIs that cross multiple thresholds (e.g., green to yellow or white

to red). Evaluate and characterize these results to allow timely indication of

declining performance.

**Criteria:** Expect few occurrences, with a stable or declining trend.

**Goals Supported:** Ensure Safety, Risk-Informed, Ensure Effectiveness



**Analysis:** There were four occurrences of PIs crossing multiple thresholds during this assessment period:

- 1) D.C. Cook Units 1 and 2 moved from Green to Yellow for the Alert and Notification System PI with the 2Q07 data submittal.
- 2) Salem 1 moved from Green to Yellow for the Mitigating System Performance Index Emergency AC Power PI with the 4Q07 data submittal.
- 3) Browns Ferry 1 moved from Green to Yellow for the Unplanned Scrams with Complications PI with the 4Q07 data submittal.

This metric did not meet its criteria based on the fact that three sites (four units) crossed multiple thresholds in CY 2007. This is an increasing trend from previous years. Also, each site crossed multiple thresholds due to a different PI, indicating that this is not PI dependent or an isolated occurrence. The assessment period data indicates an adverse trend that requires additional attention and close monitoring from the staff.

PI-4 PI Program Provides Insights to Help Ensure Plant Safety

**Definition:** Survey external and internal stakeholders asking whether the PI Program

provides useful insights to help ensure plant safety.

**Criteria:** Expect a low number of negative comments, with a stable or declining trend.

Goals Supported: Ensure Safety, Ensure Effectiveness, Risk-Informed

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Responses from the public questioned the PI Program due to the declining

number of greater-than-green PIs but not a proportional decrease in the number of reactors receiving heightened NRC attention. A public response also stated that PIs only provide some insight to plant safety, referring to the Davis Besse plant; and that it was difficult to show that some PIs are actually related to plant

safety.

The State respondent agreed that PIs are based on operating safety but questioned whether the indicators in the PI program are periodically "re-set" to reflect the differences in observed occurrences and the current expectations.

Industry responses were generally in agreement that the PI program does provide useful insights to ensure plant safety but noted that efforts should continue to better risk-inform the PIs and improve the level of insight they provide. Industry responses also stated that initial indications of the enhancements (e.g., Scrams with Complications and Mitigating Systems Performance Index PIs) made, improved the program but need more time to fully assess the effectiveness of the changes.

The staff recognizes the need to improve the PI Program to provide more meaningful indications of declining plant performance. This metric does not meet its established criteria because public and State respondents gave overall feedback that the PIs do not provide an adequate indication of declining safety performance. Also, the internal survey conducted last year noted adverse trends regarding the NRC staff's view of the public's confidence in the PI program. Many comments noted that threshold for the PIs is too high to identify declining performance and, because the PIs are almost always green, they do not enhance public confidence.

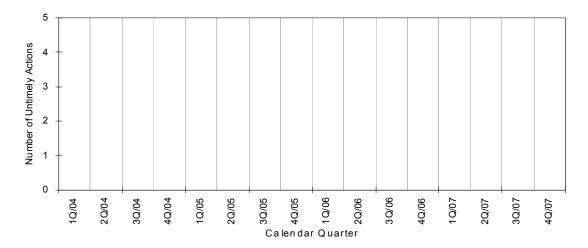
#### PI-5 Timely PI Data Reporting and Dissemination

**Definition:** Within 5 weeks of the end of each calendar quarter, track (count) late PI postings

on the NRC's external Web site. Also note the number of late submittals from

licensees that did not meet the 21-day timeliness goal.

**Criteria:** Expect few occurrences, with a stable or declining trend.



Goals Supported: Ensure Effectiveness, Ensure Openness, Predictable

**Analysis:** There were no late postings on the NRC's external Web site.

One power plant provided a PI submittal that was late; however, the submittal was made within a few days after the date required by NEI 99-02, "Regulatory Assessment Performance Indicator Guideline." The late PI data submittal from the licensee did not impact the NRC's ability to post the results on the web page in a timely manner.

The criteria for this metric have been met because there have been no late PI data postings on the NRC's external web site since the inception of the ROP.

PI-6 Stakeholders Perceive Appropriate Overlap Between the PI Program and

**Inspection Program** 

**Definition:** Survey external and internal stakeholders asking if appropriate overlap exists

between the PI program and the inspection program.

**Criteria:** Expect a low number of negative comments, with a stable or declining trend.

**Goals Supported:** Ensure Effectiveness, Ensure Safety, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Public response varied regarding proper overlap between the PI and inspection

programs. One public response again noted the ineffectiveness of the PI program. Another public response agreed that appropriate overlap exists but

gave an example of not enough overlap to catch some safety issues.

The State response indicated that there appears to be appropriate overlap but

was not knowledgeable enough to ascertain the information.

Industry comments noted appropriate overlap overall, but some stated concerns with possible double counting of inspection findings and performance indicator

results that arise from the same event.

There was a decline in the number of negative comments from CY 2006:

therefore this metric has been met.

PI-7 Clarity of Performance Indicator Guidance

**Definition:** Survey external and internal stakeholders asking if NEI 99-02, "Regulatory

Assessment Performance Indicator Guideline," provides clear guidance

regarding performance indicators.

**Criteria:** Expect a low number of negative comments or examples of interpretation issues,

with a stable or declining trend in the number of negative comments received.

Goals Supported: Understandable, Ensure Openness, Objective

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Public responses stated that combined with the FAQ process, the guidance was

clear.

State stakeholders generally felt that the guidance was clear but that it would be more appropriate for the licensees to provide comments on effectiveness of the

PI guidance.

Industry respondents commented that the PI guidance is clear and that the FAQ process and the appeal process have proven to be responsive and effective in

addressing questions and resolving issues.

Based on comments, MSPI guidance is effective but needs improvement, which

will be an on-going process. There was a decline in the number of negative

comments from CY 2006; therefore this metric has been met.

PI-8 PI Program Identifies Performance Outliers In an Objective and Predictable

Manner

**Definition:** Survey external and internal stakeholders asking if the PI program can effectively

identify performance outliers based on risk-informed, objective, and predictable

indicators.

**Criteria:** Expect a low number of negative comments, with a stable or declining trend.

**Goals Supported:** Risk-Informed, Objective, Predictable

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Public comments state that the PI program could and does identify outliers when

not manipulated by the industry and is also dependent on the type of

performance and indicator.

State stakeholders noted that the PI program provides the ability to track and trend, but as noted earlier, the thresholds may need periodic re-examination.

Industry comments were favorable about the PI program, particularly regarding MSPI because it is risk based and incorporates unavailability and unreliability. Industry responses also encouraged continued development of risk informed

elements for future and existing Pls.

The staff is continually reviewing and revising PIs to provide meaningful indications of plant performance and to better identify performance outliers. There was a decline in the number of negative comments from CY 2006;

therefore this metric has been met.

#### IP-1 Inspection Findings Documented In Accordance With Requirements

**Definition:** Audit inspection reports in relation to program requirements (IMC 0612, "Power

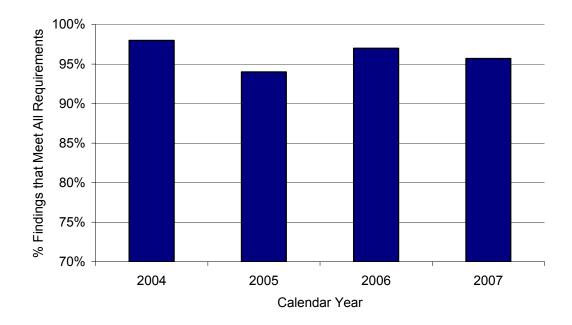
Reactor Inspection Reports") for documenting green findings, greater-than-green findings, and violations. Report the percentage of findings that meet the program

requirements.

**Criteria:** Expect a stable or improving trend in the percentage of findings documented in

accordance with program requirements.

Goals Supported: Objective, Risk-Informed, Predictable



#### Analysis:

The staff audited integrated inspection reports from each branch and a number of team inspection reports from each region. Of the 618 inspection reports issued in CY 2007, 43 were included in this audit. The percentage of findings documented in accordance with IMC 0612 requirements for the sample audited was 96 percent. Overall the data confirms that a stable trend has been maintained since 2004.

#### IP-2 Number of Feedback Forms per Document

**Definition:** 

Count the number of feedback forms received for each program document each quarter. Use a histogram to chart the number of documents for which feedback forms were received. Analyze the trend in number of feedback forms received for individual program documents, taking into account major changes in the documents and feedback forms unrelated to the document itself.

Criteria:

Expect a declining trend in the number of feedback forms received for program documents.

Goals Supported: Understandable, Predictable, Objective

**Analysis:** 

The staff received 123 feedback forms in CY 2007. Approximately 66 percent of all feedback forms received during this period relate to issues in the areas of:

- (1) IMC 0612, Power Reactor Inspection Reports (22 Forms), 17%;
- (2) IMC 0305, ROP Assessment (10 Forms), 8%;
- (3) IMC 0609, Significance Determination Process (SDP) including all Appendices (9 Forms), 7%;
- (4) IMC 1245, Qualification Program for the Office of Nuclear Reactor Regulation Programs (19 forms), 15%
- (5) IMC 2515, Appendix D, Plant Status (8 forms), 7%
- (6) IP 71151, Performance Indicator Verification (5 Forms), 4%.
- (7) IP 95001, Inspection for One or Two White Inputs in a Strategic Performance Area (8 forms), 7%

Of the 123 feedback forms received this calendar year, the staff resolved 101 forms (82%). Overall, the staff has resolved 191 feedback forms from a total of 223 feedback forms (86%). The total number of feedback forms includes 100 feedback forms brought forward from CY 2006 of which, the staff closed 90. There were 32 feedback forms open at the end of CY 2007. This number included the remaining 22 forms received this year and 10 forms carried over from CY 2006.

Changes to the ROP Feedback Process implemented at the beginning of FY 2007 had mixed results. The use of the NRR Work Planning and Control Center for resource usage and timeliness tracking proved cumbersome. It was of no benefit for feedback resolution within the NRR inspection branches or that required coordination outside of the Office of Nuclear Reactor Regulation, and only limited benefit within the other NRR branches. The process will be refined to allow flexibility in the use of the work planning process where appropriate.

Significant progress in staff performance and responsiveness to feedback occurred as expected in the past year, with a reduction in the backlog of unresolved feedback from 57 forms in 2006 to 32 forms this year. In the coming year, the staff will concentrate on implementation of agreed upon resolutions of feedback. There is a current backlog of 106 resolved feedback forms that

identified changes to be implemented to ROP documentation. Of these 106 pending changes, 22 forms exceed a year in age.

Feedback submission demonstrates a downward trend for IMC 0612, IMC 0609 and its appendices, and IMC 1245 which meets the metric criterion. The submission rate for IMC 0305 was slightly positive. The trend for IMC 0305 submissions is driven in part by a high number of submissions in early 2006 and as the result of safety culture enhancements. A review of the submissions showed that they included 3 duplicates that were administratively cancelled and the majority were the result of solicitation of feedback on the assessment process following the end of cycle assessment meetings for the prior calendar year. This trend is not statistically significant, particularly when corrected for duplicate submissions. A slight upward trend is noted for feedback linked to IP 71151. Feedback for this procedure focused primarily on the interpretation of performance indicators or their guidelines, NEI 99-02, rather than on the procedure itself. Discounting these submissions, a downward trend in feedback is present. A slight upward trend was noted in feedback received for IMC 2515 Appendix D. This trend is statistically insignificant, in part due to the low number of feedback forms received. The feedback rate for IP 95001 showed an upward trend, driven by the submission of seven forms in the first quarter of CY 2007. Field implementation of IP 95001 at the Palisades plant had resulted in the lead inspector submitting almost all of the IP 95001 feedback forms. The feedback provided value added recommendations to further enhance IP 95001 and related inspection guidance documents. The upward trend does not appear to be statistically significant due to the otherwise low number of submissions.

With the few exceptions detailed above, the feedback received on the ROP exhibited downward trends. For those exceptions, drivers not impacting the understandability, predictability, and objectivity of the documentation were present.

#### IP-3 Completion of Baseline Inspection Program

**Definition:** Annual completion of baseline inspection program.

**Criteria:** Defined as per IMC 2515, "Light-Water Reactor Inspection Program - Operations

Phase."

Goals Supported: Ensure Safety, Predictable, Ensure Effectiveness

**Analysis:** All four regions completed their baseline inspections in CY 2007. Each region

documented completion of the program in a memorandum to the Division of Inspection and Regional Support in NRR. These memoranda can be found in ADAMS under ML080430029 (Region I), ML080770153 (Region II), ML080450429 (Region III), and ML080730456 (Region IV). As in the 2006 inspection cycle, all regions completed their baseline inspections in 2007 with the allocated regional

resources.

In addition, NSIR documented completion of the security baseline inspection

program (ML080390446), but this document is not publicly available.

#### IP-4 Inspection Reports Are Timely

**Definition:** Obtain RPS data on the total number of reports issued and the number issued

within timeliness goals as stipulated in IMC 0612, "Power Reactor Inspection

Reports."

**Criteria:** Expect 90 percent of inspection reports to be issued within program's timeliness

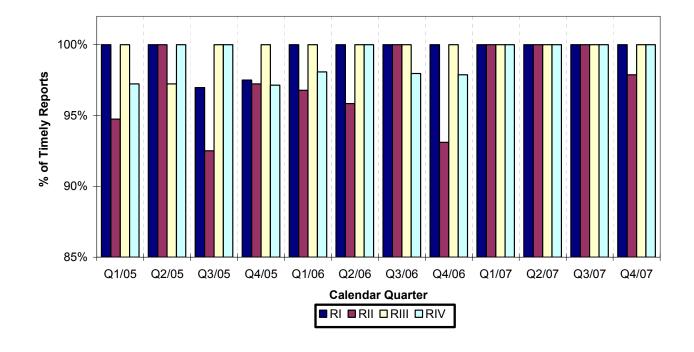
goals.

**NOTE:** For inspections not conducted by a resident inspector, inspection completion is

normally defined as the day of the exit meeting. For resident inspector and integrated inspection reports, inspection completion is normally defined as the

last day covered by the inspection report.

**Goals Supported:** Ensure Effectiveness, Ensure Openness, Predictable



**Analysis:** A total of 618 inspection reports were issued during CY 2007. Regions met or

exceeded the inspection report timeliness goal of 90 percent in each quarter throughout the year. In CY 2007, only one inspection report out of the 618 failed

to meet the timeliness requirement per IMC 0612.

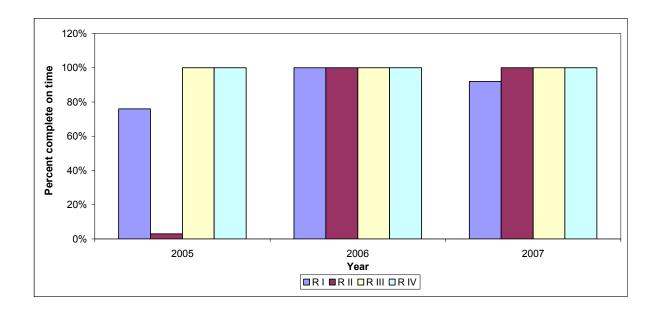
#### IP-5 Temporary Instructions (TIs) Are Completed Timely

**Definition:** Audit the time to complete TIs by region. Compare the completion status in RPS

to TI requirements. Report by region the percentage of TIs closed within goals.

**Criteria:** Expect all TIs to be completed within TI requirements.

Goals Supported: Ensure Effectiveness, Ensure Safety, Predictable



#### Analysis:

Only one TI had an expiration date in 2007 (2515/154, "Spent Fuel Material Control and Accounting at Nuclear Power Plants," expired on August 31, 2007). Although the TI was completed at 98 percent of the sites within timeliness goals, one site completed the TI approximately 3 weeks past the expiration date. The delay was necessary after the licensee identified issues affecting their readiness for the inspection, which in turn delayed NRC inspection efforts. The staff is considering changing the criteria from 100 percent complete to 95 percent in the next revision of IMC 0307 to allow for conditions beyond the staff's control.

#### IP-6 Public Communication Is Accurate

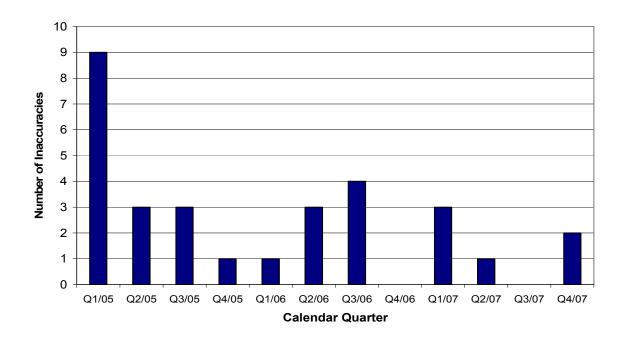
**Definition:** Each calendar quarter, sample information on the NRC's external (public) Web

site and count the number of times and reasons for regions changing PIMs or

inspection reports (i.e., inaccuracy, new information).

**Criteria:** Expect few inaccuracies, with a stable or declining trend.

Goals Supported: Ensure Openness, Ensure Effectiveness, Understandable



Analysis:

There were few inaccurate postings of PIM entries or inspection reports on the web identified during CY 2007. No region had more than two inaccurate postings per quarter. The total number of inaccurate postings has been steadily declining since CY 2005. This metric met program expectations.

IP-7 Inspection Reports Are Relevant, Useful, and Written in Plain Language

**Definition:** Survey external and internal stakeholders asking whether the information

contained in inspection reports is relevant, useful, and written in plain English.

**Criteria:** Trend average level of agreement.

Goals Supported: Ensure Effectiveness, Understandable, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** External stakeholders generally agreed that the inspection reports were clearly

written and useful. However, the following comments for improvement were

provided:

More effort is spent documenting the details of an issue then in providing
insights as to why the issue is a concern. The true nature, magnitude, and
risk significance of the issue is often not addressed. When cross-cutting
aspects are assigned to findings and/or violations, the inspection report
documentation sometimes lacks a clear explanation as to how the crosscutting aspect is a significant contributor to the cause of the issue.

 Current inspection reports are so concise that it is often difficult to obtain useful information. It is even more difficult to read last year's report and figure out exactly what inspectors observed.

The NRC staff is evaluating these comments and plans to address them in the consolidated response.

### IP-8 Inspection Program Effectiveness and Adequacy in Covering Areas Important to Safety

**Definition:** Survey external and internal stakeholders asking whether the inspection program

adequately covers areas that are important to safety and is effective in identifying

and ensuring the prompt correction of performance deficiencies.

**Criteria:** Trend average level of agreement.

Goals Supported: Ensure Safety, Ensure Effectiveness, Risk-Informed

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** The responses were positive with the following comments for improvement:

 The ROP realignment is only one method for monitoring the inspection program and may not be the most efficient. Obtaining, reviewing, and acting on continuous feedback from licensees and inspection personnel can be more timely and efficient. Maintaining a self-critical, questioning position regarding the value of inspection activities is a key to keeping the inspection program vital and robust.

- Enhancements could be made to the inspection program to make better
  use of the NRC generic communications program (such as technical
  questions identified during inspections that involve development of new
  regulatory positions). Two examples of inspection issues with generic
  implications are 1) manual actions for response to fires and 2)
  assessment of post-fire safe shut down equipment. A process is needed
  to ensure early stakeholder involvement in the identification and
  resolution of inspection issues that potentially have generic implications.
- Performance Identification and Resolution (PI&R) inspections fail to
  effectively identify and mitigate performance deficiencies in a timely
  manner. It would be beneficial if NRC inspectors would follow-up on
  findings from the NRC Inspection Program as opposed to examining a
  large sample of corrective action reports. By using NRC inspection
  findings to define the sample, the audit automatically begins with
  evidence of corrective action program problems.
- Inspectors need more freedom to inspect rather than follow prescribed quotas of items. The program does not ensure corrective action but only that an item is entered into the corrective action program.

The NRC staff is evaluating these comments and plans to address them in the consolidated response.

#### IP-9 Analysis of Baseline Inspection Procedures

**Definition:** 

Annually, review each baseline inspection procedure to determine its effectiveness and contribution to the overall effectiveness of the baseline inspection program. The objectives of the review are: (1) to determine if changes in scope, frequency, or level of effort are needed based on recent experience, (2) to determine if a change to the estimated hours for completion is needed, (3) to define or change what constitutes minimum completion of each inspectable area, if needed, and (4) to critically evaluate all of the inspectable areas together along with the PI program to ensure that the inspectable areas are adequately monitored for safety performance. In addition, a more detailed review and realignment of inspection resources will be performed at least biennially in accordance with Appendix B to this Chapter. The focus of this effort is to adjust existing inspection resources to improve the effectiveness of the inspection program in identifying significant licensee performance deficiencies.

Criteria:

None; trend only. Summarize and evaluate the individual inspection procedure reviews and propose program adjustments as necessary to address noted inefficiencies. Provide basis for any meaningful increase or decrease in procedure scope, frequency, or level of effort as a result of the review.

Goals Supported: Ensure Effectiveness, Ensure Safety

Analysis:

The inspection program staff conducted the biennial review of the ROP inspection procedures to ensure most effective application of inspection resources. This biennial review process is part of the ROP self-assessment program and it is described in Appendix B to Inspection Manual Chapter 0307, "ROP Alignment Process." As part of this review, the NRR staff established a working group to gauge the effectiveness of each of the inspection procedures. The working group examined the inspection resources used for each procedure in the Initiating Events, Mitigating Systems, Barrier Integrity, Occupational Radiation Safety, and Public Radiation Safety cornerstones to determine whether appropriate inspection resources were being applied. Inspection procedures in the EP and Security cornerstones were not reviewed as part of the ROP realignment effort in CY 2007. The review considered inspection results over a 3-year period (CY 2004 through CY 2006).

The working group consisted of NRR staff and each of the four regions. The results of the ROP working group were shared with senior NRR management from Division of Inspection and Regional Support (DIRS), NRR, and with regional Division Directors from Division of Reactor Projects and Reactor Safety. Majority positions were reached by senior regional managers for those inspection procedures which were determined to require a revision to the inspection program element of the ROP. The staff made changes affecting inspection scope and frequency to 12 baseline inspection procedures and implemented the revised baseline inspection program beginning in CY 2008. The results of the 2007 ROP realignment process can be obtained from the Agencywide Documents Access and Management System (ADAMS ML073020593). The staff plans to perform the next ROP realignment in CY 2009, and the baseline

inspection program will reflect any changes resulting from that effort starting in CY 2010.

In addition to the detailed ROP realignment process, the staff performed its annual evaluation of the inspection procedures in fiscal year (FY) 2007 to determine whether any additional improvements to the baseline inspections were warranted based on inspection findings over the most recent FY. The staff also performed a best practices review of the problem identification and resolution inspection procedure (IP 71152, "Identification and Resolution of Problems"). The purpose of the review was to help ensure consistent implementation of the procedure and to identify potential effectiveness and efficiency improvements. The staff made recommendations and identified some potential changes as a result of these reviews that will be evaluated in CY 2008.

# SDP-1 The SDP Results Are Predictable and Repeatable and Focus Stakeholder Attention on Significant Safety Issues

**Definition:** 

Annually, audit a representative sample (up to four per region) of inspection findings against the standard criteria set forth in IMC 0609, "Significance Determination Process," and its appendices. To the extent available, samples should include potentially greater-than-green findings that were presented to the Significance Determination Process/Enforcement Review Panel (SERP). Findings should contain adequate detail to enable an independent auditor to trace through the available documentation and reach the same significance color characterization.

Criteria:

The target goal is at least 90% are determined to be predictable and repeatable. Any SDP outcomes determined to be non-conservative will be evaluated and appropriate programmatic changes will be implemented.

**Goals Supported:** Ensure Safety, Risk-Informed, Predictable

Analysis:

An independent review of 32 inspection findings from the 2006-2007 inspections was performed by the Division of Risk Assessment in NRR. The representative sample was doubled to ensure accuracy of the results given the metric was revised and this review is its first use. Each sample represents a finding assessed using the risk-informed process detailed in IMC 0609 Appendix A, "Determining the Significance of Reactor Inspection Findings for At-Power Situations." All samples were determined to include adequate detail to be predictable and repeatable and the documentation of the final determination and evaluation of the green findings was clear, and is consistent with past reviews. Performance during this assessment period meets program expectations.

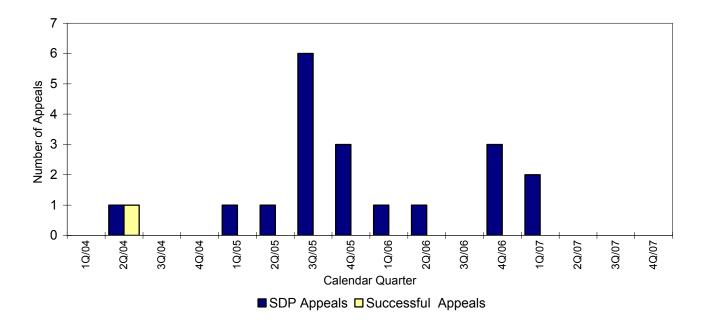
#### SDP-2 SDP Outcomes Are Risk-Informed and Accepted by Stakeholders

**Definition:** Track the total number of appeals of final SDP results.

**Criteria:** Expect zero appeals of SDP significance that result in a final determination being

overturned across all regions. All successful appeals will be assessed to determine causal factors and to recommend process improvements.

**Goals Supported:** Risk-Informed, Objective, Predictable



#### Analysis:

There were two appeals by licensees of SDP significance outcomes. Both of the appeals were aimed at findings of low to moderate safety significance (i.e., WHITE findings). One appeal was for an emergency preparedness issue and the other for a reactor mitigating system. In each case, the final outcome was unchanged.

Performance during this assessment period meets program expectations based on no successful appeals of SDP significance determinations that result in a final greater-than-green being overturned.

SDP-3 Inspection Staff Is Proficient and Find Value in Using the SDP

**Definition:** Survey internal stakeholders using specific quantitative survey questions that

focus on training, effectiveness, and efficiency.

**Criteria:** Expect either a stable or an increasingly positive perception of the SDP process

over time.

Goals Supported: Ensure Effectiveness, Understandable, Risk-Informed

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

### SDP-4 SDP Provides an Objective and Understandable Regulatory Response to Performance Issues

**Definition:** Survey external and internal stakeholders asking if the SDP results in an

objective and understandable regulatory response to performance issues.

**Criteria:** Expect stable or increasingly positive perception of the SDP over time.

**Goals Supported:** Understandable, Objective, Predictable

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

Analysis: Generally, the respondents believe the SDP is a useful tool to quickly determine

a plant's status in specific oversight areas and that it is generally risk-informed; however, it remains complex requiring one to be an expert on the SDP process. More than half of the industry respondents stated that the Radiation Protection, Security, and Emergency Preparedness SDPs are subjective and deterministic,

not based on risk or actual effect thresholds and may result in exaggeration of

actual risk.

For reactor SDPs supported by SPAR modeling, the industry representatives believe the methodology minimizes realistic factors such as operator actions and tends to utilize hypothetical worst case assumptions. While the steps in an SDP decision are well outlined, they feel that the bases and rationale for these steps are often neither objective nor predictable and can be inconsistent with industry norms for probabilistic risk assessment (PRA) decisions. They further believe that the NRC should abandon the use of SPAR and use the Licensee's Regulatory Guide 1.200 compliant PRA models to support the SDP process.

In the first half of 2007, the NRC met with representatives from industry and other stakeholders, in a series of public meetings, to discuss the industry proposal to use industry PRA analyses in lieu of the NRC risk assessment tools. The NRC reviewed the industry proposal and concluded that the reactor oversight process required the NRC to maintain independence by evaluating the significance of findings and not just review the results of licensee's assessment. At present, the industry has not uniformly implemented a standardized approach to performing risk analysis that would ensure uniform application across the spectrum of industry PRA models. In this regard, the NRC's use of the SPAR models, together with the ongoing development of guidance on conducting Phase 3 risk assessments, commonly referred to as the risk assessment standardization project (RASP), ensures greater uniformity in the agency's regulatory assessments.

This metric does not meet its criteria based on a stable negative perception over the past seven years of ROP implementation.

## SDP-5 The Resources (Direct Charges and Support Activities) Expended Are Appropriate

**Definition:** Track the percentage of total resource expenditures attributed to SDP activities

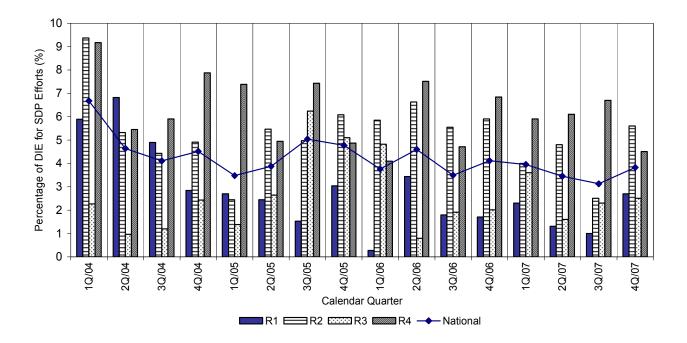
to determine the effort expended by the regions in completing SDP evaluations

as a percentage of the total regional direct inspection effort.

Criteria: Total SDP expenditures should not exceed 10 percent of the total regional direct

inspection effort (DIE) with a stable or declining trend.

**Goals Supported:** Ensure Effectiveness, Predictable



Analysis: Regional expenditures associated with SDP evaluations remain stable and below

the target goal. Performance during this assessment period meets program

expectations.

#### SDP-6a Final Significance Determinations Are Timely

**Definition:** 

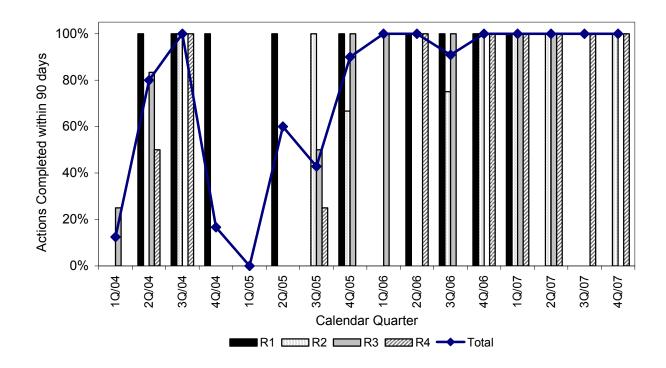
Conduct a quarterly audit of RPS data to identify the total number of inspection items finalized as greater than green that were under review for more than 90 days since:

- (1) the date of initial licensee notification of the preliminary significance in an inspection report, or
- (2) the item was otherwise documented in an inspection report as an apparent violation pending completion of a significance determination.

Criteria:

At least 90% of all SDP results that are counted per the criteria above should be finalized within 90 days. All issues greater than 90 days will be assessed to determine causal factors and to recommend process improvements.

**Goals Supported:** Ensure Effectiveness, Ensure Openness, Predictable



Analysis:

Timeliness of final significance determinations increased from 96% for FY 2006 to 100% for FY 2007. Performance during this assessment period met and exceeded program expectations.

#### SDP-6b Final Significance Determinations Are Timely

#### **Definition:**

Conduct a quarterly audit of issues that were assessed by the Significance Determination Process/Enforcement Review Panel (SERP) to identify the total number of inspection items finalized as green or greater-than-green that were under review for more than 90 days since:

- (1) the date of initial licensee notification of the preliminary significance in an inspection report or otherwise documented in an inspection report as an "AV" pending completion of a significance determination, or
- (2) the date the item was presented to the SERP for review.

#### Criteria:

At least 90% of all SDP results that are counted per the criteria above should be finalized within 90 days on average and 100% in 180 days. All issues greater than 180 days will be assessed to determine causal factors and to recommend process improvements.

NOTE: This metric is being piloted as a potential replacement for the existing SDP timeliness metric.

**Goals Supported:** Ensure Effectiveness, Ensure Openness, Predictable

Analysis:

No issues assessed by the SERP exceeded 90 days under review. The average age of all the SDP results that were presented to the SERP during FY 2007 issues was 62 days.

#### SDP-7 SDP Results Are Communicated Accurately to the Public

**Definition:** Each calendar quarter, track the number of inspection findings that are

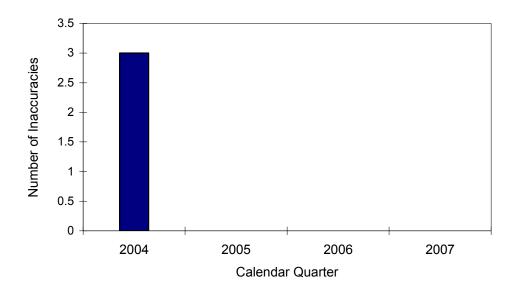
inaccurately communicated to the public (color of findings is inaccurately reported) by auditing the inspection findings summary information available on the NRC Web. The detailed review will include item type, significance characterization, enforcement action status, and text descriptions of greater-than-

green inspection findings prior to release to external stakeholders.

Criteria: The target goal is zero inaccuracies, with a stable or declining trend. All

inaccuracies must be addressed.

Goals Supported: Ensure Openness, Understandable, Ensure Effectiveness



**Analysis:** During the current assessment period no inaccuracies were identified. Performance during this assessment period meets program expectations.

AS-1 Actions Are Determined by Quantifiable Assessment Inputs (i.e., Pls and SDP Results) and are Commensurate With the Risk of the Issue and Overall Plant Risk.

**Definition:** Audit all assessment-related letters and count the number of deviations from the

Action Matrix. Evaluate the causes for these deviations and identify changes to

the ROP, if any, to improve the guidance documents.

**Criteria:** Expect few deviations, with a stable or declining trend.

Goals Supported: Objective, Predictable, Ensure Openness

Analysis:

There have been a total of fifteen (15) deviations from the Action Matrix since the beginning of the ROP in CY 2000. Three of these deviations occurred in CY 2007. This metric meets its criteria based on the unique circumstances addressed by the deviations, and the approval of only one new deviation. The staff's evaluation of the 2007 Action Matrix Deviations concluded the following:

- (1) On October 28, 2005, and renewed on December 11, 2006, and December 19, 2007, the Executive Director for Operations (EDO) approved deviation memorandums to provide heightened NRC oversight at the Indian Point Energy Center. The staff intends to continue to closely monitor the licensee's actions in CY 2008 to address issues associated with onsite ground-water contamination characterization and mitigation and with the ANS, including implementation and testing of the replacement ANS that Entergy is installing in response to the Energy Policy Act of 2005. The actions for the Indian Point Energy Center represent a customized approach that considers factors beyond each unit's Action Matrix categorization. This approach is consistent with underlying concepts of IMC 0305.
- (2) On May 16, 2005, and renewed in July 2006 and August 2007, the EDO approved deviation memorandums to provide heightened NRC oversight at Davis-Besse. The staff intends to continue monitoring the licensee's efforts to sustain improved plant performance following resolution of the long-standing underlying problems that culminated in a red finding associated with the severe wastage that was discovered on the reactor vessel head. As noted in last year's self-assessment, the staff revised IMC 0305 to allow the regional offices to use additional followup actions for plants that are exiting the IMC 0350 process. The programmatic changes made as a result of this deviation could prevent the need for similar deviations in the future.

(3) The NRC issued a deviation memorandum in November 2007 to address security-related concerns at the Peach Bottom site. The security-related finding also had a documented cross-cutting aspect in the area of safety conscious work environment (SCWE). The NRC has taken several actions in evaluating the licensee's scope of effort and progress in addressing the SCWE cross-cutting aspect and inattentiveness issues. The NRC actions included augmented inspection teams and a confirmatory action letter. These NRC actions provide the regulatory framework to monitor the company's progress in addressing

security-related and SCWE issues at Peach Bottom until the next performance assessment. The deviation memorandum documents the company's agreement to take certain actions in response to inattentiveness on the part of some security officers. The company's actions include detailed briefings to security force personnel on acceptable behavior; round-the-clock supervisory oversight of security activities, and keeping the NRC informed of the status of the Peach Bottom transition from a contractor security force to one that is run by Exelon. The commitments in the letter will remain in effect until the NRC has reviewed Exelon's root cause analysis of the security program issues, the company's corrective actions and implementation schedule, and the company's method for assessing the effectiveness of the corrective actions. As a result of these issues, a temporary instruction has been developed to inspect the transition of contract security force to proprietary security force. The staff continues to evaluate this deviation for impact on the ROP and will consider program improvements based on the lessons learned from the ongoing evaluation.

AS-2 The Number and Scope of Additional Actions Recommended as a Result of the Agency Action Review Meeting (AARM) Beyond Those Actions Already Taken Are Limited

**Definition:** Review the results of the Agency Action Review Meeting (AARM).

**Criteria:** Few additional actions, with a stable or declining trend.

Goals Supported: Understandable, Predictable, Objective

**Analysis:** The AARM was held on April 18, 2007, in Bethesda, Maryland. The participants

confirmed the appropriateness of agency actions for Point Beach 1 and 2, Perry, and Palo Verde. The participants did not recommend any additional actions beyond those already taken or planned. The next Agency Action Review

Meeting is scheduled for early May 2008.

## AS-3 Assessment Program Results (Assessment Reviews, Assessment Letters and Public Meetings) Are Completed in a Timely Manner

**Definition:** Track the number of instances in which timeliness goals stipulated in IMC 0305,

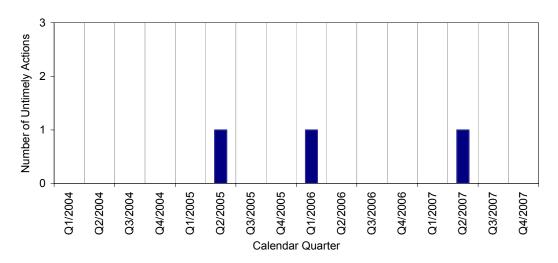
"Operating Reactor Assessment Program," were not met for: (1) the conduct of quarterly, mid-cycle, and end-of-cycle reviews; (2) the issuance of assessment

letters; and (3) the conduct of public meetings.

**Criteria:** Expect few instances in which timeliness goals were not met, with a stable or

declining trend.

Goals Supported: Ensure Effectiveness, Ensure Openness, Predictable



**Analysis:** Timeliness goals for the following activities are as follows:

- (1) quarterly reviews within 5 weeks of the end of quarter
- (2) mid-cycle reviews within 7 weeks of the end of the 2<sup>nd</sup> guarter
- (3) end-of-cycle reviews within 7 weeks of the last guarter
- (4) issuance of assessment letters within 2 weeks of the quarterly review, within 9 weeks of the mid-cycle review, and within 9 weeks of the end-of-cycle review
- (5) conduct of public meetings within 16 weeks of the end of the assessment period.

4Q/2007: All quarterly reviews and assessment follow-up letters were completed within timeliness goals.

3Q/2007: All mid-cycle review meetings, mid-cycle letters and quarterly assessment reviews were conducted within timeliness goals. Additionally, all public meetings were completed within timeliness goals.

2Q/2007: All quarterly assessment reviews were completed within timeliness goals. One assessment follow-up letter was *not* completed within timeliness goals. All public meetings were completed within timeliness goals.

1Q/2007: All end-of-cycle reviews, assessment letters, quarterly assessment reviews and assessment follow-up letters were completed within timeliness goals. Additionally, all public meetings were completed within timeliness goals.

## AS-4 The NRC's Response to Performance Issues Is Timely

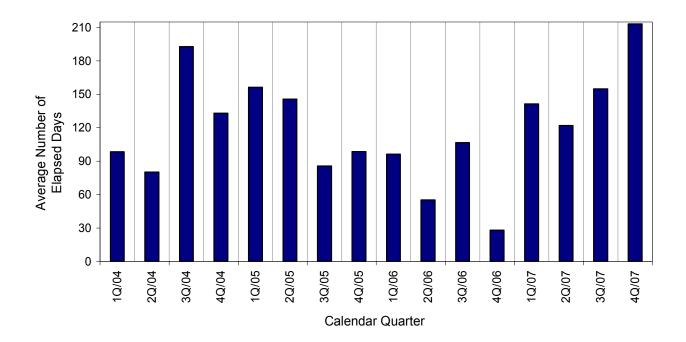
**Definition**: Count the number of days between issuance of an assessment letter discussing

an issue of more than very low safety significance and completion of the supplemental inspection (by exit meeting date, not issuance of the inspection

report).

**Criteria:** Expect a stable or declining trend.

Goals Supported: Ensure Safety, Ensure Effectiveness



Comments: The data represents an average timeliness for the supplemental inspections

completed in each region in any given quarter.

**Analysis:** Data collected to date indicates an increasing trend regarding the elapsed time

between the issuance of an assessment letter and the completion of the corresponding supplemental inspection over previous years. The staff will continue to monitor the adverse trend for this metric. However, the delays in performing the follow-up inspections were often due to the licensee not being ready for the inspection. As a result, the staff is considering revising this metric to better account for licensee readiness, replacing it with something more

meaningful, or deleting it altogether.

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AS-5 NRC Takes Appropriate Actions to Address Performance Issues

**Definition:** Survey external and internal stakeholders asking whether the NRC takes

appropriate actions to address performance issues for those plants outside the

Licensee Response Column of the Action Matrix.

**Criteria:** Expect stable or improved perception.

**Goals Supported:** Ensure Safety, Ensure Effectiveness, Understandable

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Public response agreed that the NRC takes appropriate actions to address

performance deficiencies, safety culture problems, and cross-cutting aspects, but suggested that the NRC actions with enforcement actions involving civil penalties

were not appropriate (i.e., too lenient).

The industry and States agreed that actions taken by the NRC for plants outside of the licensee response column have been appropriate. There are industry concerns that the NRC action in accordance with the Action Matrix is consistent for single White findings, but appears less consistent for more complex issues.

Industry responses also stated that there appears to be a degree of

inconsistency between regions with the assignment of cross-cutting aspects.

The overall level of external stakeholder satisfaction in this area was favorable

and similar to previous years.

AS-6 Assessment Reports Are Relevant, Useful, and Written in Plain Language

**Definition:** Survey external and internal stakeholders asking whether the information

contained in assessment reports is relevant, useful, and written in plain English.

**Criteria:** Expect stable or improved perception of the relevance, usefulness, and

understandability of assessment reports.

**Goals Supported:** Understandable, Ensure Effectiveness, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** One public interest group stated that the assessment reports contain too much

boilerplate information preventing substantive insights about performance at individual sites. This has been a similar comment made in previous years but has been refuted by the NRC staff in previous consolidated responses to

stakeholder comments.

The industry representatives and the State respondent agreed that the information contained in assessment reports is relevant, useful, and written in plain English. Industry responses noted concern with cross-cutting issues.

The overall level of external stakeholder satisfaction in this area was favorable

and similar to previous years.

# AS-7 Degradations in Plant Performance Are Gradual and Allow Adequate Agency Engagement of the Licensees

**Definition**: Track the number of instances each quarter in which plants move more than one

column to the right in the Action Matrix (as indicated on the Action Matrix

Summary).

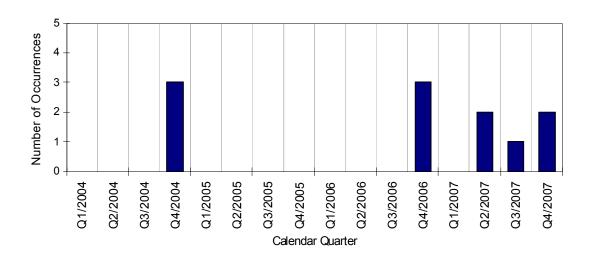
**Criteria:** Expect few instances in which plant performance causes a plant to move more

than one column to the right in the Action Matrix. Provide a qualitative

explanation of each instance in which this occurs. Expect a stable or declining

trend from the first-year benchmark.

Goals Supported: Risk-informed, Ensure Safety, Predictable



## Analysis:

D.C. Cook Units 1 and 2, moved from the licensee response column to the degraded cornerstone column in 2Q/2007 due to a yellow PI in the EP cornerstone for the Alert and Notification System.

Farley Unit 1, moved from the licensee response column to the degraded cornerstone column in 3Q/2007 due to a White PI in the Mitigating System Cornerstone for Cooling Water System issues and a parallel White Performance Indicator finding in the Mitigating System cornerstone regarding breaker failures.

Salem Unit 1 moved from the licensee response column to the degraded cornerstone column in 4Q/2007 due to a yellow PI in the Mitigating System Cornerstone for Emergency AC Power System.

Browns Ferry Unit 1 moved from the licensee response column to the degraded cornerstone column in 4Q/2007 due to a yellow PI for Unplanned Scrams in the Initiating Events Cornerstone.

This metric did not meet its criteria because 4 distinct sites (5 units) moved two or more columns to the right in the Action Matrix in CY 2007, an increasing trend from previous years. Also, each site moved more than one column to the right due to a different PI or inspection finding. The assessment period data indicates a short-term adverse trend as only one site had moved two or more columns in the Action Matrix since the fourth quarter of 2004. The staff will assess the data and engage with the industry to better understand the root causes to determine if this is actual degradation in licensee performance or something else.

## AS-8 Perceived Effectiveness of Safety Culture Enhancements to ROP

**Definition:** Survey external and internal stakeholders asking whether the ROP safety culture

enhancements help in identifying licensee safety culture weaknesses and

focusing licensee and NRC attention appropriately.

**Criteria:** Expect stable or improved perception over time. Trend average level of

agreement.

Goals Supported: Ensure Effectiveness, Ensure Safety

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

Analysis: Several questions were included in the external survey related to the ROP safety

culture enhancements. Public comments stated that the CY 2007 survey questions related to safety culture were not written in plain English and difficult to understand. The majority of the industry responses stated that it was too soon to judge whether the ROP inspection and assessment safety culture enhancements have helped to focus licensee and NRC attention on performance issues

associated with safety culture.

Several respondents provided input on IP 95003, "Supplemental Inspection for Repetitive Degraded Cornerstones, Multiple Degraded Cornerstones, Multiple Yellow Inputs, or One Red Input." Industry commented that a focused lessons learned review should be performed on IP 95003 following its recent implementation at Palo Verde. The industry commented that the IP overemphasizes the potential contribution of safety culture and that it raises questions about industry use of safety culture surveys.

Four industry responders stated that the cross-cutting aspects need to be reconsidered as only a few are being used and that some of the cross-cutting aspects are too broadly defined. Two industry responses stated the minimum number of inspection findings that are necessary to develop a theme and a substantive cross-cutting issue should be increased from the current four findings. Comments were also received about inspection report documentation of cross-cutting aspects, for example two industry responses stated that inspection report documentation should be updated if the assigned cross-cutting aspect changes.

Based on the relatively short period of time that the enhanced ROP has been in effect, a trend has not been established. A more meaningful assessment of the metric criterion will be achieved in follow-on years as the safety culture changes have been implemented for a longer period. The safety culture performance criterion is considered not applicable at this time. As part of the ROP safety culture lessons learned evaluation described in Enclosure 2 to this SECY, the staff is considering comments from the external survey. The NRC staff will

repeat the sub questions related to safety culture in future external surveys, using plain English, in an effort to clarify them.

Metric Criterion Met: Not applicable at this time.

O-1 Stakeholders Perceive the ROP to Be Predictable and Objective

**Definition:** Survey external and internal stakeholders asking if ROP oversight activities are

predictable (i.e., controlled by the process) and reasonably objective (i.e., based

on supported facts, rather than relying on subjective judgment).

**Criteria:** Expect a stable or increasing positive perception over time.

**Goals Supported:** Objective, Predictable, Ensure Effectiveness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** The State respondent noted that ROP oversight activities did provide consistency

and objectivity.

Public responses were somewhat contrary. One public respondent stated that the objectivity of the ROP was one of the biggest problems and that subjectivity is beneficial. However, another public response stated that due to the lack of public documentation of the basis for decisions, they perceived the ROP

activities and outcomes to be arbitrary.

The responses from licensees were generally in favor of the predictability and objectivity aspects in the ROP. Some utilities and their representatives indicated that the substantive cross-cutting issues, cornerstones not closely tied to risk analysis, double counting, and characterizing minor issues are the most unpredictable and subjective areas that need improvement. It was also suggested that Manual Chapter 0612 Appendix E, be a "living" document (i.e., continually add and update examples).

Comments received were similar to previous years. As a result of the positive perception, this metric has been met.

O-2 Stakeholders Perceive the ROP to Be Risk-informed

**Definition:** Survey external and internal stakeholders asking if the ROP is risk-informed, in

that actions and outcomes are appropriately graduated on the basis of increased

significance.

**Criteria:** Expect stable or increasingly positive perception over time.

Goals Supported: Risk-Informed, Ensure Effectiveness, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** The State and public responses agreed that the ROP is risk-informed. However,

one public response stated the ROP being risk-informed was a program weakness, mentioning it was overly risk-informed. The public respondent also

expressed reservations over the direction of the risk informed process.

Industry representatives agreed that some of the cornerstones were risk-informed, but for other cornerstones, the risk basis for the thresholds were more

subjective and less predictable (as opposed to graduated on the basis of increased significance). Industry responses were favorable but suggested areas

for improvement.

Overall, respondents believe the ROP is generally risk informed.

O-3 Stakeholders Perceive the ROP to Be Understandable

**Definition:** Survey external and internal stakeholders asking if the ROP is understandable

and if the processes, procedures, and products are clear and written in plain

English.

**Criteria:** Expect stable or increasingly positive perception over time.

Goals Supported: Understandable, Ensure Effectiveness, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** In general, industry, State and public respondents stated that the ROP is

understandable and that products are written in clear and plain English. One response from the public disagreed, citing the MSPI as a specific example. Licensees expressed difficulty in following some of the complex documents without the appropriate technical background. Similar to previous years, the SDP

is recognized as the most complex portion of the ROP and the NRC staff is

continuously looking to improve this aspect of the program.

O-4 Stakeholders Perceive That the ROP Provides Adequate Regulatory Assurance That Plants Are Operated and Maintained Safely

**Definition:** Survey external and internal stakeholders asking if the ROP provides adequate

regulatory assurance, when combined with other NRC regulatory processes, that

plants are being operated and maintained safely.

**Criteria:** Expect stable or increasingly positive perception over time.

Goals Supported: Ensure Safety, Ensure Effectiveness, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** The two public respondents did not believe the ROP (combined with other

regulatory processes) assured safe plant operations.

Industry and State stakeholders believe the ROP, combined with other regulatory processes assures plants are operated and maintained safely. The State response was more positive than others in previous years, noting that the ROP provides a substantial framework for ensuring that safety remains a primary

focus for the licensees.

There is an increasingly positive industry and State perception of the ROP maintaining safety. Public comments were similar to previous years' surveys.

O-5 Stakeholders Perceive the ROP to Be Effective, Efficient, Realistic, and Timely

**Definition:** Survey external and internal stakeholders asking whether the ROP is effective,

efficient, realistic, and timely.

**Criteria:** Expect stable or increasingly positive perception over time.

**Goals Supported:** Ensure Effectiveness, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** The State respondent did not comment citing lack of experience.

The two public responses ranged from "no" to all portions of the question, to perhaps effective, yes efficient, likely realistic and not timely. Responses were

comparable to previous years.

In general, utility respondents believe that the ROP is effective, efficient, realistic and timely in comparison to previous programs. The utility stakeholder suggested areas for improvement such as the SDP, the appeal process, and for plants coming out of extended outages. Some industry responses noted that the NRC should consider reevaluating the frequency of certain inspections and the mid-cycle assessment for plants in the licensee or regulatory response column.

This metric met its criteria with a stable perception over time.

O-6 Stakeholders Perceive That the ROP Ensures Openness

**Definition:** Survey external and internal stakeholders asking if the ROP ensures openness in

the regulatory process.

**Criteria:** Expect stable or increasingly positive perception over time.

Goals Supported: Ensure Openness, Ensure Effectiveness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** External stakeholders generally acknowledged that the ROP ensures openness

in the regulatory process, but both public and utility stakeholders expressed

some concerns and noted that further improvements could be made.

For example, one public response stated that there is not enough openness in

the ROP because the bases for decisions are not made publicly available.

Utility stakeholders reiterated that the ROP is generally a very open process, but pointed out the security process, the TIA process and the SDP activities as

specific areas that should be more open and allow for greater stakeholder input.

The State respondent indicated that the ROP promotes openness but cannot

ensure openness because the process can be intimidating for members of the

public.

This metric met its criteria with a stable perception over time.

O-7 Opportunities for Public Participation in the Process

**Definition:** Survey external and internal stakeholders asking if there are sufficient

opportunities for the public to participate in the process.

**Criteria:** Expect positive responses or an improving trend.

Goals Supported: Ensure Openness, Ensure Effectiveness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Public respondents do not believe there is sufficient opportunity for public

participation. One response noted the Category 2 designation of the meetings, stating members of the public can only make comments during inopportune times. The public respondent also noted that the agenda for the monthly public meetings is not detailed enough. However, based on prior feedback from external stakeholders, the staff added significant detail to the agenda in the meeting notices so that potentially interested stakeholders could determine prior to the meeting whether topics of interest to them were planned topics of

discussion. The State respondent did not comment citing lack of experience.

Industry stakeholder responses were very positive. They acknowledged the ample opportunities for public participation such as monthly public meetings at NRC headquarters, annual public meetings conducted in the reactor

communities, annual assessment meetings at each site, and annual ROP

surveys.

The NRC is always striving to ensure public participation and will continually provide sufficient and timely opportunities for members of the public to do so.

O-8 Stakeholders Perceive the NRC to Be Responsive to its Inputs and Comments

**Definition:** Survey external and internal stakeholders asking if the NRC is responsive to the

public's inputs and comments on the ROP.

**Criteria:** Expect positive responses or an improving trend.

**Goals Supported:** Ensure Openness, Ensure Effectiveness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Public stakeholders feel that the NRC is responsive to input and feedback but

sometimes with only select public interest groups. A public response stated that there is a perception on the part of the public that the NRC does not value their input. The State respondent did not comment citing lack of experience. Industry stakeholders believe that the NRC is responsive to inputs and comments, noting the published response to the 2006 ROP survey and encouraged continued

responses for future surveys.

Overall, stakeholder satisfaction as reported in the survey responses for the ROP was generally favorable and consistent. The NRC will continue to publish its consolidated response to the external surveys and encourage public input and

feedback.

O-9 Stakeholders Perceive That the ROP is implemented as Defined

**Definition:** Survey external and internal stakeholders asking if the ROP has been

implemented as defined by program documents.

**Criteria:** Expect stable or increasingly positive perception over time.

**Goals Supported:** Predictable, Understandable, Ensure Openness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** In general, most external stakeholders (including utilities, State, and public

respondents) believe the ROP is being implemented as defined.

One public stakeholder did not agree that the NRC implemented the ROP as

defined citing post-renewal inspections as an area for improvement.

Utility stakeholders only concern in this area was the potential lack of consistency

when informal communications and decisions are made between inspectors and

NRR staff.

This metric met its criteria with mostly positive comments and a stable positive

perception over time.

O-10 Stakeholders Perceive That the ROP Does Not Result in Unintended Consequences

**Definition:** Survey external and internal stakeholders asking if the ROP results in unintended

consequences.

**Criteria:** Expect stable or increasingly positive perception over time.

Goals Supported: Ensure Safety, Ensure Effectiveness

**NOTE:** There was no internal survey in CY 2007 consistent with its biennial frequency as

defined by IMC 0307. The next survey to internal stakeholders will be conducted

during the fourth quarter of CY 2008.

**Analysis:** Responses from Public stakeholders were contrary. One public response stated

the ROP does not result in unintended consequences. Another public response

stated yes, it does result in unintended consequences because licensees

emphasize some PIs at the suffering of others.

The State stakeholder did not comment citing lack of experience.

Utility responses stated that generally the ROP does not result in unintended consequences but one industry respondent cited a specific example of when the ROP did result in unintended consequences, therefore additional guidance

should be issued for clarification.

This metric met its criteria with a stable positive perception over time.

O-11 Analysis of NRC's Responses to Significant Events

**Definition:** Review reports from incident investigation teams (IITs) and augmented

inspection teams (AITs) to collect lessons learned regarding ROP programmatic deficiencies (i.e., did the baseline inspection program inspect this area? Did the SDP accurately characterize resultant findings?). IITs already have the provision to determine NRC program deficiencies. AITs will be reviewed by NRR/DIRS to

identify any weaknesses.

**Criteria:** Expect no major programmatic voids.

**Goals Supported:** Ensure Safety, Ensure Effectiveness

**Analysis:** No IITs were conducted during the 2007 ROP cycle. One AIT was conducted in

CY 2007. Staff review of the AIT did not identify any program weaknesses or voids. Two feedback forms were received for IP 93800 regarding resource allocation for AIT follow-up inspections. The feedback forms resulted in a

revision to the IP.

## O-12 Analysis of Inspection Hours and Resource Expenditures

**Definition:** 

Annually, collect and analyze resource data (e.g., direct inspection effort, preparation/documentation, plant status hours) for Baseline, Supplemental/Plant-Specific, and Safety Issues Inspections, and other ROP activities.

Criteria:

- (1) Significant deviations are not expected on an annual basis. Explore reasons for any deviations that may be evident.
- (2) Track and trend resource usage for the baseline inspection program and supplemental/plant-specific inspections. Analyze causes for any significant departure from established trend.
- (3) Track and trend resource usage for preparation, documentation, and other ROP activities, and assess the effects on budgeted resources.

NOTE: This metric is intended primarily for tracking and trending resource usage for the ROP. The results are used to improve the efficiency and effectiveness of the ROP and to make management and budget decisions. A detailed ROP resource analysis is included in the annual ROP self-assessment Commission paper.

Goals Supported: Ensure Effectiveness, Predictable

Analysis:

Overall staff effort for ROP related activities in Fiscal year (FY) 2007 increased 2.3% compared with FY 2006.

Baseline inspection hours increased in 2007 primarily due to increased direct inspection effort with a corresponding increase in baseline inspection preparation and documentation. The inspection procedures (IP) that account for the bulk of the increase are: IP 7111121, Component Design Bases Inspection; IP 71152, Identification and Resolution of Problems; and IP 71153, Follow-up of Events and Notices of Enforcement Discretion. The direct inspection effort for the baseline inspections funded by the Office of Nuclear Security and Incident Response (NSIR) in FY 2007 remained essentially unchanged from the FY 2006 levels.

There was a noticeable overall decrease in plant-specific inspections in FY 2007 compared with FY 2006. The decrease was evident in all the components of plant-specific inspections. However, only resource data for the period September 24, 2006, through September 22, 2007, is included and several significant inspections took place after this time period. Those expenditures will be captured in the FY 2008 results and the NRC staff expects a significant increase in resources spent.

An increase in effort related to Generic Safety Inspections (GSIs) reflects the growing activity in this area. The GSIs are typically one time inspections of specific safety issues with significant variability in effort possible from year to year.

## O-13 Analysis of Resident Inspector Demographics and Experience

### **Definition:**

Annually, collect and analyze data in order to determine the relevant inspection experience of the resident inspector (RI) and senior resident inspector (SRI) population. The following four parameters will be measured and analyzed for both RIs and SRIs to ensure that the NRC maintains a highly qualified resident inspection staff:

- (1) NRC time the total time the individual has accumulated as an NRC employee.
- (2) <u>Total resident time</u> the total time the individual has accumulated as an RI or SRI.
- (3) <u>Current site time</u> the total time the individual has spent as an RI or SRI at the current site.
- (4) Relevant non-NRC experience the total time the individual has gained relevant nuclear power experience outside of the NRC. Examples of relevant non-NRC experience are operation, engineering, maintenance, or construction experience with commercial nuclear power plants, naval shipyards, U.S Department of Energy facilities, or the U.S. Navy nuclear power program.

#### Criteria:

None; trend only. Provide reasons for any meaningful increase or decrease in these resident demographic metrics.

NOTE: This metric is intended primarily for tracking and trending resident inspection experience. The results are used to make any necessary modifications to the RI and/or SRI programs in order to attract and retain highly qualified inspectors to the respective programs. A detailed resident demographic and staffing analysis, including additional graphs, data, and analysis for these resident demographic metrics, is included in the annual ROP self-assessment Commission paper.

Goals Supported: Ensure Safety, Ensure Effectiveness

### **Analysis:**

<u>Analysis of 2007 RI Group</u> - RI demographic data for 2007 (see Tables 1 and 3 and Figure 1) indicate that the RI turnover rate increased resulting in a decrease of both total resident time and current site time. Although the turnover rate has increased and current site time and total resident time numbers are down, the RIs continue to maintain a high level of experience.

During 2007, 33 of 72 RIs left an RI position (46 percent). Of the 33 RIs who left, 13 were promoted to SRI positions, 13 were either promoted or laterally reassigned to a region or headquarters, 3 retired, and 4 resigned from the NRC. Table 1 tracks the RI turnover from 2003 to 2007.

Table 1
Resident Inspector Turnover

	2003	2004	2005	2006	2007
Promoted to SRI	14	3	10	11	13
Promoted / reassigned	12	3	9	2	13
Retired	1	0	2	1	3
Resigned	0	0	2	0	4
Total	27	6	23	14	33
Turnover Rate	38%	8%	32%	20%	46%

The RI turnover rate for 2007 increased by 26 percent from 2006. Region II had 13 RI vacancies, and Regions I and IV both had 8 RI vacancies. Even though a significant portion of the RIs were promoted to an SRI position (40 percent), an equal share of the RIs were either promoted or reassigned outside the RI program.

Nationally both the total resident time and current site time are less than two years. The national median value (NMV) for total resident time decreased 20 percent from 2006 to 2007. This decrease offset the gradual 20 percent increase in the NMV for total resident time from 2003 to 2006.

The national data from 2003 to 2007 shows that the RIs have maintained an average of 10 years relevant non-NRC experience and 4 years of NRC time. This demonstrates that the RIs continue to maintain a high level of experience despite the high turnover rates in recent years. There were 18 new RIs in 2007 and they had an average of 10 years of relevant non-NRC experience and 3 years of NRC time. This shows that the new RIs that are filling open positions have a substantial amount of nuclear experience. The staff is considering combining the NRC time and relevant non-NRC experience data to reflect overall nuclear experience in the next revision to IMC 0307.

NOTE: The RI demographics data in 2008 will reflect the addition of five new RIs from November to December 2007.

The staff was directed in the staff requirements memorandum dated June 14, 2007 to evaluate the recruitment, training, and development of the RI program to confirm that the human resources are adequate to meet changing needs. The staff collected the following information:

RI Recruitment -The regions recruit inspectors externally from universities, service academy career conferences, job fairs, the Nuclear Safety Professional Development Program, U.S. Navy and shipyards, the nuclear power industry, and corporate engineering firms. The regions also recruit internally by posting RI vacancies within the region or nationwide. Due to the high turnover rate in 2007, the regions have had difficulty filling RI vacancies and have implemented various recruitment strategies to fill the open positions. The staff will continue to evaluate the RI recruitment strategies in 2008.

<u>RI Training</u> - In 2007, the regions qualified 23 individuals in accordance with Inspection Manual Chapter (IMC) 1245, "Inspector Qualification Program." Currently, 41 individuals are in the IMC 1245 qualification program, and 38 are projected to receive their inspector qualification in 2008. Overall, the inspector training program in the regions is well established and continues to produce qualified inspectors.

<u>RI Development</u> - RIs and SRIs continue to develop professionally by filling rotational assignments and participating in team inspections, training opportunities, inspector seminars, and knowledge transfer sessions.

<u>Analysis of 2007 SRI Group</u> - SRI demographic data for 2007 (see Tables 2 and 4 and Figure 2) indicate that the SRI turnover rate was high resulting in a national decrease in current site time from 2006 to 2007. Although there was a national drop in current site time, the NMV for total resident time and relevant non-NRC time has increased annually since 2003.

In 2007, 17 of 66 SRIs left the program (26 percent). Of those 17, 7 were promoted, 7 were laterally reassigned to headquarters or a region, 1 retired, and 2 resigned from the NRC. Table 2 tracks the SRI turnover from 2003 to 2007.

Table 2
Senior Resident Inspector Turnover

	2003	2004	2005	2006	2007
Promoted	7	0	5	7	7
Reassigned	6	3	4	7	7
Retired	1	2	1	1	1
Resigned	0	0	0	1	2
Total	14	5	10	16	17
Turnover Rate	21%	8%	15%	24%	26%

The SRI turnover rate in 2007 (26 percent) is about the same as in 2006; however, it has been increasing since 2004. The high national SRI turnover rate directly affected the NMV for current site time. In 2007, all four regions' current site time was low. This decrease offset the gradual national current site time increase from 2003 to 2006. Although the national current site time numbers have decreased, the NMV for total resident time and relevant non-NRC experience has been gradually increasing since 2003.

The staff was directed in the staff requirements memorandum dated June 14, 2007, to consider ways to enable SRIs to be promoted and still remain within the RI program. A task force of staff from the Office of the Executive Director for Operations and the Deputy Regional Administrators is currently assessing RI program retention issues and will provide the Deputy Executive Director for Regulatory Programs with recommendations and potential solutions.

## <u>Conclusions</u> - The staff concluded the following:

- The combined RI and SRI turnover rate increased significantly in 2007 creating a complex regional human resource allocation issue which directly affected the site staffing metric (O-14).
- The NMVs for RI total resident time and for SRI current site time decreased from 2006 to 2007 as a result of the high turnover rates.
- The overall experience levels (NRC time and relevant non-NRC time) of RIs and SRIs have remained high as indicated by the national data since 2003 (see Figures 1 and 2).

Table 3
Resident Inspectors

	2003	2004	2005	2006	2007
NRC Time	4.13	3.42	3.36	4.04	4.25
Total Resident Time	1.99	2.00	2.31	2.39	1.87
Current Site Time	1.00	1.85	2.25	2.23	1.85
Relevant non- NRC Experience	10.00	10.00	10.63	10.75	10.38

Figure 1

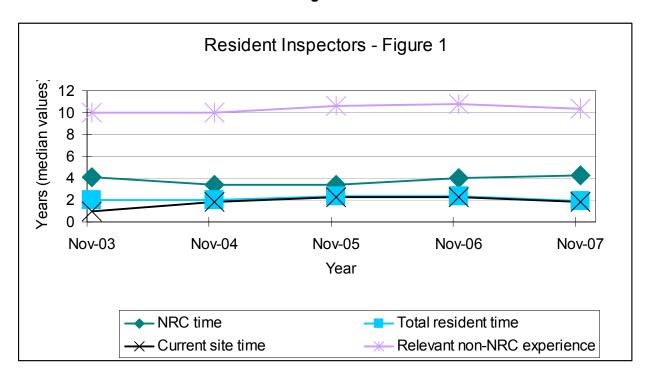
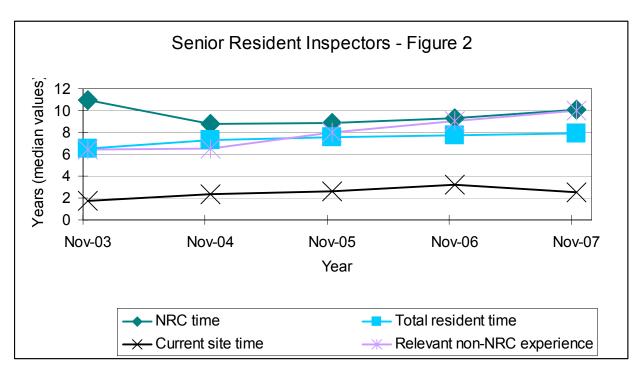


Table 4
Senior Resident Inspectors

	2003	2004	2005	2006	2007
NRC Time	11.00	8.80	8.84	9.28	10.11
Total Resident Time	6.48	7.32	7.54	7.77	7.93
Current Site Time	1.76	2.31	2.63	3.21	2.52
Relevant non- NRC Experience	6.42	6.55	7.96	9.08	10.04

Figure 2



## O-14 Analysis of Site Staffing

**Definition:** Annually, collect and analyze data in order to measure the permanent inspector

staffing levels at each of the reactor sites for both RIs and SRIs in order to evaluate the agency's ability to provide continuity of regulatory oversight.

**Criteria:** The criterion is set at 90% program-wide. Any single site that falls below 90% will be individually evaluated. Provide reasons for any meaningful increase or

decrease in the inspector staffing level at reactors sites.

NOTE: Inspectors assigned to the site permanently or through a rotation with a minimum duration of 6 weeks shall be counted. Inspectors on 6 week or longer rotational assignments will be identified as such. Inspectors assigned to the site for less than six weeks will not be counted, but should be indicated as such. Additionally, the regions shall indicate sites where permanently assigned resident or senior resident inspectors are away from the site for an extended period of time (one continuous time period which is greater than 6 weeks). Only inspectors who have attained at least a basic inspector certification status, as defined by Appendix A to Inspection Manual Chapter 1245, shall be counted.

Data will indicate number of days a qualified resident and senior resident inspector are permanently assigned to the site during the year divided by the number of days in the year. Number of days spent on training; meetings away from the site; participation in team inspections; leave; or other temporary duties (e.g. acting for branch chiefs in his/her absence) will not be counted against the metric unless the absence exceed 6 continuous weeks.

Goals Supported: Ensure Safety, Ensure Effectiveness

Analysis:

The criterion for the metric is 90 percent program-wide. In 2007 the average site staffing for all the regions was 96 percent, with each region exceeding 90 percent. However, nine sites were below the 90 percent mark. Of these nine sites, eight were between 84 and 89 percent, and one site was 74 percent. The site that had 74 percent site staffing had an RI that retired from the NRC in June of 2007. Qualified inspectors filled the vacancy temporarily for periods of less than 6 weeks for the duration of calendar year. In 2005 three sites did not meet the criterion of 90 percent and in 2006 only one site was below 90 percent.

As a result of the high turnover rate in 2007, the regions were presented a significant challenge in providing continuity of regulatory oversight at the affected sites. Two regions were able to meet the 90 percent site staffing goal at all of the sites; however, this was a significant burden on the regional staff and management. The RI and SRI vacancies were filled by inspectors on extended rotations, resulting in complex regional human resource allocation issues. Additionally, inspection resources were provided by other regions and headquarters enabling the region to fill the openings for 6 weeks or longer and meet the metric. The other two regions experienced difficulty in staffing the vacant RI and SRI positions for extended periods resulting in 9 sites falling below the 90 percent site staffing goal. In 2007, the sites that fell below the 90 percent

mark were dealing with an RI or SRI who left the site permanently (either by retirement, resignation, or transfer to a regional or headquarters position). To support the site inspection efforts, the regions provided qualified inspectors to the sites where inspectors were needed for periods of less than 6 weeks. Because these periods were less than 6 weeks, the site was recorded as not continuously staffed during this timeframe. However, at no time did these sites remain without qualified inspectors to support the required inspection efforts.

O-15 Analysis of ROP Training and Qualifications

**Definition:** Annually, evaluate the implementation of IMC 1245, "Qualification Program for

the Office of Nuclear Reactor Regulation Programs," particularly as it pertains to

ROP implementation.

**Criteria:** None; trend only. Summarize and evaluate the training accomplished over the

previous year and propose program improvements as necessary to address

noted concerns.

NOTE: This metric is intended primarily for tracking and trending the

effectiveness of the ROP training and qualifications programs. A discussion of

training effectiveness is included in the annual ROP self-assessment

Commission paper.

Goals Supported: Ensure Effectiveness, Ensure Safety

**Analysis:** The staff continued to improve the initial and continuing inspector training

programs in order to produce and maintain well-qualified, competent inspectors. Improvement actions identified by the staff were reviewed in accordance with the ROP feedback process and the improvements incorporated into inspection standards, as appropriate. The staff updated safety culture training for

inspectors and performed further training at the regional counterpart and security inspector counterpart meetings. The staff also developed and implemented computer-based training on the new Unplanned Scrams with Complications performance indicator. Additional discussion of training effectiveness is included in the inspection program evaluation in Enclosure 1 of the CY 2007 ROP Self-

Assessment Commission paper.

O-16 Analysis of Regulatory Impact

**Definition:** Annually, collect and analyze licensee feedback and develop a summary of

regulatory impact forms that are critical of the ROP.

**Criteria:** None; trend only. Summarize and evaluate the feedback received and propose

program improvements as necessary to address common concerns.

NOTE: This metric is intended primarily for tracking and trending regulatory impact. A detailed regulatory impact summary is included in the annual ROP

self-assessment Commission paper.

Goals Supported: Ensure Effectiveness, Ensure Safety

**Analysis:** The staff receives and evaluates feedback from licensees on an annual basis as

part of the regulatory impact process. The regulatory impact process was established in 1991 based on Commission direction to develop a process for

obtaining feedback from licensees and reporting the feedback to the

Commission. Over the past year, the staff received feedback from 68 reactor licensees on 139 issues, which was a decrease over 2006. The comments fell into two main categories, formal communication with licensees and inspector performance. Of the comments received, 86 percent were favorable and 14 percent were unfavorable. A summary of the feedback received, the staff's

evaluation, and the proposed improvement actions are provided in Enclosure 4 of

the CY 2007 ROP Self-Assessment Commission paper.