## UNITED STATES NUCLEAR REGULATORY COMMISSION

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# BRIEFING ON THE STATUS OF SUBSEQUENT LICENSING RENEWAL PREPARATIONS

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WEDNESDAY,

APRIL 26, 2017

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### ROCKVILLE, MARYLAND

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The Commission met in the Commissioners' Hearing Room at the Nuclear Regulatory Commission, One White Flint North, 11555 Rockville Pike, at 8:58 a.m., Kristine L. Svinicki, Chairman, presiding.

**COMMISSION MEMBERS:** 

KRISTINE L. SVINICKI, Chairman

JEFF BARAN, Commissioner

STEPHEN G. BURNS, Commissioner

ALSO PRESENT:

ANNETTE VIETTI-COOK, Secretary of the Commission

MARGARET DOANE, General Counsel

### **EXTERNAL PANELISTS PRESENT:**

- PAUL AITKEN, Manager, Second License Renewal,

  Dominion Resources, Surry Power Station, Units 1 and 2
- SHERRY BERNHOFT, Program Manager, Long Term
  Operations, Electric Power Research Institute
- MICHAEL GALLAGHER, Vice President, License Renewal
  Projects, Exelon Generation Co., LLC, Peach
  Bottom Atomic Power Station, Units 2 and 3
- DAVID LOCHBAUM, Director, Nuclear Safety Project,
  Union of Concerned Scientists
- RICHARD REISTER, Director, Light Water Reactor

  Sustainability Program, U.S. Department of Energy
- S. JASON REMER, Director, Plant Life Extension,

  Nuclear Energy Institute

#### NRC STAFF PRESENT:

VICTOR MCCREE, Executive Director for Operations
STEVE BLOOM, Office of Nuclear Regulatory Regulation
MICHELE EVANS, Deputy Director, Officer of Nuclear
Regulatory Regulation

ALLEN HISER, Senior Technical Advisor for License
Renewal Aging Management, Division of License
Renewal, Office of Nuclear Reactor Regulation

JOHN PARILLO, Office of Nuclear Regulatory Regulation

JEFFREY POEHLER, Office of Nuclear Reactor Regulation

BRIAN THOMAS, Director, Division of Engineering,
Office of Nuclear Regulatory Research
GEORGE WILSON, Director, Division of License
Renewal, Office of Nuclear Reactor Regulation

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	Good	morning	and	we	will	begin	our	topical	meeting	on
subsequent lice	ense rer	newal prep	oarati	ons.	Со	uld I as	k the	panelis	ts for the	first
panel to please	take th	eir seats?	•							

Good morning. Again, thank you all for being here today and to our panelists on both panels for presenting to the Commission this morning.

We need for the Commission to entertain a discussion on the status of issues related to subsequent license renewal application and the NRC staff's preparedness to review an application. Since the Commission's action on this matter substantively in 2014, the staff has continued to work on the Guidance and other documents that will guide their reviews of subsequent license renewal applications and the Commission. I think that this is very timely that we meet today to hear a status of the staff's work and also to hear from a panel of outside participants, who will provide us with perspectives both on how that work has developed and also other status updates and information relevant to the topic.

Before we begin, I would ask if my colleagues have any opening comments. Okay, thank you.

With that, let us then turn to the External Panel. I will simply just do an introduction of each of the panelists and then we can begin. And if you all can hand off to each other in the order in which you are seated, that would be very helpful. Thank you.

So we will begin with Richard Reister, director of the Light Water Reactor Sustainability Program from the U.S. Department of Energy.

We will also hear from Sherry Bernhoft, the Program Manager

Τ	of Long-Term Operations at the Electric Power Research Institute.
2	Following Sherry, we will hear from Jason Remer, Director of
3	Plant Life Extension of the Nuclear Energy Institute.
4	Next we will hear from Michael Gallagher, Vice President
5	License Renewal Projects of Exelon Generation Company.
6	Following that we will hear from Paul Aitken, Manager I'm
7	sorry if I have mispronounced anyone's names. I hope you will correct me
8	when you begin your talk. Paul is the Manager of Second License Renewal for
9	Dominion Resources.
10	And we will than hear from Mr. David Lochbaum, Director
11	Nuclear Safety Project Union of Concerned Scientists.
12	Thank you all again for participating in this morning's meeting.
13	Let's begin with Mr. Reister.
14	MR. REISTER: Good morning and thank you for inviting me
15	here today to discuss the Department of Energy Office of Nuclear Energy's
16	program on Light Water Reactor Sustainability or LWRS, as we say.
17	The LWRS program began in 2009 with the recognition that the
18	U.S. Government had a strategic interest in the long-term operation of our
19	existing fleet of nuclear power plants in support of energy security, which
20	includes attributes such as fuel supply, diversity, and grid reliability, and to help
21	avoid or at least defer the high cost of building replacement capacity.
22	The LWRS program has partnered with industry and the
23	Nuclear Regulatory Commission to closely coordinate our research needs and
24	share costs. Next slide. I guess oh, there we go.
25	This graph illustrates the current and projected capacity of
26	existing nuclear power plants with 40-, 60-, and 80-year licenses, accounting for

publicly announced shutdowns and ongoing new construction. Assuming all plants receive 60-year licenses, the slope of retirements of very steep, about five plants per year in the 2030 to 2035 time frame.

The rate of retirements that would occur without subsequent license renewal would clearly present a significant challenge to our electricity supply, infrastructure, and our energy security. Next slide.

The key objectives of the light water reactor sustainability program is to extend the life of our existing fleet of operating plants but this can only be accomplished if they remain safe and economic to operate. The LWRS research program has four main technical focus areas: materials, aging, and degradation; advanced instrumentation, information, and control system technologies for improving performance and economics through better use of digital technologies; risk-informed safety margin characterization, which develops advanced modeling and simulation capabilities to better manage safety margins; and reactor safety technologies, which examines severe accident progression and supports improvements to severe accident management guidelines.

I will focus on the materials area in more detail, since it directly relates to the licensing topic. Next slide.

The LWRS program conducts research that is generally longer term, higher risk than what industry typically performs, along with materials issues typically a focus of licensing. The LWRS address long-term viability or economics of continued plant operation. We believe this federal program, by reducing uncertainty, helps create an environment for industry to make the long-term investments necessary to keep these plants operating safely and efficiently.

As plants age, the potential for material degradation occurs.

Our research is focused on being proactive by conducting research on relevant materials. This includes materials harvested from plants and materials that undergo accelerated aging.

When possible, mechanistic aging models are developed and validated. The results of our research are used by industry and the regulator to inform an update of their Aging Management Programs. More than half of our research funds are devoted to materials research.

For material degradation issues, we cover these five elements: the collection of high-quality data on observed degradation; determining the underlying physical phenomena causing the degradation; development of mechanistic models, based on the physical mechanisms rather than just empirical trends; developing improved monitoring techniques by taking advantage of new sensors and advanced data analysis; and investigating mitigation strategies to limit degradation and, if needed, economic repair or replacement with materials less susceptible to degradation. Next slide.

The expanded material degradation assessment was a joint DOE and NRC effort to determine knowledge gaps, along with our associated risk significance. This assessment helped us working with EPRI and NRC to determine the key areas of focus of our research in reactor pressure vessels, core internals and piping, concrete, and cables. We believe we made significant progress in our understanding of degradation in these areas. No technical showstoppers to long-term operation have been identified through our research.

Our research continues to improve our understanding and reduce uncertainties which can be used to further improve Aging Management

Programs. Next slide.

The embrittlement of reactor pressure vessels has been generally understood and managed for many years. Our research is being conducted on test specimens that have undergone accelerated irradiation at test reactors and will soon include samples collected from a shutdown Zion plant.

Based on research to date, U.S. reactor pressure vessel embrittlement or hardening of the steel is following predicted trends and appears to be managed well within the 60- to 80-year time period. Continued research will help develop RPV embrittlement models that go beyond 80 years. Next slide.

Our research on core internals has focused on conducting microstructural evaluations of both laboratory materials irradiated in test reactors and the examination of materials taken from operating reactors. We're using the results of this research to develop and validate physics-based models of key degradation mechanisms. These models can be used by industry to better understand and manage core internal degradation as plants age. Next slide.

We are conducting research on the effects of fatigue and thermal aging on reactor piping materials. Current fatigue models use empirical methods based on testing conducted in air, rather than water. We have developed a thermal fatigue model for reactor pressure vessel structures which will be developed further next year to focus on the dissimilar metal weld transitions at the nozzles. Our current effort is on completing the thermal fatigue model for a PWR surge line pipe. In both models, fatigue life is based on more realistic conditions that can account for actual plant operating history.

These aging models will be validated based on operating experience within vessel materials and service materials. Next slide.

Concrete makes up the largest volume of materials used in nuclear power plants. We are conducting research on two forms of concrete degradation, irradiation and alkali-silica reaction, or ASR.

ASR effects can appear to be contradictory. When sheer reinforcement is present, the expansion due to ASR results in a pre-stressing of concrete that favors an increase in sheer capacity. In other words, it gets stronger. However, excessive ASR-induced self-pre-stressing may cause reinforcement yielding and failure. Of particular concern is the case of beams or slabs in the absence of sheer reinforcement.

Through our research, we hope to provide a better predictive capability for ASR's long-term impact on the structural integrity of large reinforced concrete structures. Generally, our research indicates that concrete structures have significant safety margins during the 60- to 80-year time period. We are also developing advanced nondestructive examination techniques for thick concrete structures. Next slide.

In the absence of either mechanical injuries or the presence of unplanned environmental conditions, cables have an excellent history of long-term performance. Our research is focused on developing a predictive aging model that will encompass most of the cable types used at U.S. plants.

Working in collaboration with the NRC's Office of Research and the Electrical Power Research Institute, we have harvested cables from operating and shutdown plants to help validate our aging models. We are also investigating promising nondestructive examination technologies that may help predict remaining useful life. Next slide.

1	In summary, DOE research is focu	used on supporting continued
2	safe and economic operation of our existing fleet.	We are proactively focusing
3	on degradation that might occur beyond 60 years	of operation.

Our research has not identified any technical showstoppers to long-term operation. We are developing improved materials monitoring techniques that will help detect degradation earlier, should it occur. We work with industry so that our research results can be used to update and enhance their Aging Management Programs.

Thank you.

CHAIRMAN SVINICKI: Sherry, please proceed.

MS. BERNHOFT: Yes, thank you. Good morning. I'm happy to be here today to provide an update on the EPRI research efforts to support the technical basis for subsequent license renewal or SLR.

Our long-term operations program actually has two purposes. The first purpose is to provide technical information to the utility plant owners, the decision-makers, so they can make informed decisions about plant long-term operations. The second purpose of our program is to support and inform the technical basis for the Aging Management Programs. Next slide, please.

I last presented to this committee on May 8, 2014. At that time, the conclusion was that the technical basis for aging management is well-established and in use. The Aging Management Programs, that is the AMPs, ensure detection of aging effects, provide the technical basis for assessment, and, as needed, corrective actions. This conclusion remains unchanged.

The overhead depicts the elements from the EPRI reports that are incorporated into utility Aging Management Programs. The NRC Generic Aging Lessons-Learned Report, or the GALL, or SLR, references over 125 of

the EPRI Technical Reports as the basis for aging management.

With this, research continues working with our research partners, such as the DOE and NRC research to further enhance our ability for aging management and assessment. This includes but is not limited to improved modeling, improved inspection technologies, advances in assessment, and evaluation methodologies and online condition monitoring. In addition, we also look for opportunity to harvest materials where that can add to the database and knowledge as well. Next slide, please.

Our research for SLR is built on EPRI's living issue management programs. This approach encompasses decades of research on reactor pressure vessels, metals, concrete, and electrical cables. A former process to review and prioritize our research needs, the collaboration with the Department of Energy, NRC Research and international research partners, coordination under NEI Initiative 03-08 -- 03-08 is the industry standard for inspection assessment and sharing of information from the Aging Management Programs; and technology transfer, which includes supporting the lead plants.

The technical basis for the reports are and will continue to be updated based on the research findings, operating experience and inspection results. Next slide, please.

As identified, there are four key technical areas for research for the subsequent license renewal. Joint research roadmaps were developed and substantial progress has been made in these four areas. Over the past several years, EPRI and DOE have interacted with the NRC Division of License Renewal and Research Staff, and held a number of deep-dive technical discussions, provided updates to the ACRS License Renewal Subcommittee and full committee, participated in a number of public meetings, and have made

our Technical Reports available to the NRC staff. Next slide, please.

For reactor pressure vessels, the aging management effect to be managed is reduction in fracture toughness of the vessel material due to irradiation. The vessel material properties are assured through periodic removal and testing of surveillance specimens to establish trend correlations. As Rich mentioned, there is a well-established embrittlement trend correlation to predict material properties out to 80 years of operation. However, we recognize additional data will enhance these trend correlations and programs have been established. For the PWRs, which have a higher fluence than the BWRs, EPRI and the industry are working together to implement the PWR Coordinate Surveillance Capsule Program and the Supplemental Program to gain additional data.

For the BWRs, there is an existing method to fulfill the surveillance requirements with an integrated program. We are working with the BWRs to extend this program for the period of SLR. Next slide, please.

For reactor pressure vessel internals, the degradation that occurs is irradiated-assisted stress corrosion cracking or IASCC of the stainless steels as a result of neutron fluence in the operating environment. The EPRI materials programs for PWRs and BWRs have published management guidelines which are referenced in the AMPs. These reports provide the basis for inspection scope, frequency, assessment, and repair and replacement criteria. These fall under the NEI Initiative 03-08 that I previously discussed.

A recent example is the fall 2016 baffle former bolt inspections.

The occurrence of IASCC was anticipated, which is why the guidance was developed. The inspection results were documented and evaluated.

Notifications have been provided and interim guidance has already been shared

with the industry, in addition to a number of technical briefings with the NRC staff and the ACRS. This demonstrates the effectiveness of the programmatic approach under NEI 03-08 and the Aging Management Programs.

In collaboration with other research parties, significant progress has been made on the crack growth rate modeling and this data has been used to develop crack growth rate criteria that is being used in the ASME Code. Harvesting of materials have also provided an opportunity for further testing and evaluation.

EPRI is leading an international project of which the NRC is a member, that harvested internal materials from the Zorita plant in Spain. Testing of the actual materials to date has confirmed the existing models are conservative for defining AISCC thresholds and crack growth rates. Next slide, please.

Moving to the area of concrete, EPRI has performed research on concrete structures for several years and has made significant progress. The main area of study for SLR is the alkali-silica reactions and the irradiation and gamma heating effects.

For ASR there is operating experience from a plant in Canada, one in the U.S., and extensive information available from the Department of Transportation.

EPRI has published reports on screening for ASR and guidance if ASR is detected. We are publishing a series of white papers and a comprehensive ASR aging management guidance later this year. Research continues on nondestructive methods to detect for ASR and modeling to improve growth rate predictions and structural assessments.

Substantial research has been completed on the area of

assessing the impacts of irradiation effects on concrete. Fundamental work was completed by the DOE program and was used to build a PWR biological shield final element analysis, which is shown on the picture. The model showed that at the projected 80-year fluence level, there is still remaining a significant structural margin and that radiation effects will not an issue for our subsequent license renewal. Next slide, please.

Under the area of electrical cables, EPRI has been conducting research for over 25 years to characterize cable insulation material aging and degradation mechanisms. The stressors are radiation and temperature for the low-voltage cables and submergence for the medium-voltage cables.

Based on our research, aging management guidelines have been published and provided to the NRC staff for their information. We also have a well-established cable users group. This group shared operating experience and best practices for cable monitoring and aging management. A detailed joint R&D roadmap was developed and activities are well underway to collect additional data on thermal and radiation age cables and improve technology for cable condition monitoring. Jointly, we have recently harvested cables from the Crystal River Unit 3 plant to confirm the as-found condition and additional aging tests are planned.

The tools and guidance exist for aging management which will provide a reasonable assurance of safety and compliance with the current licensing basis. Continuing research and condition monitoring technology will provide the tools for plant owners for their asset management programs. Next slide, please.

So in conclusion, we started off developing detailed joint research roadmaps, we worked in collaboration with our research partners, and

we have made substantial progress. Based on this work, our conclusions remain unchanged. The technical basis for aging management exists and is in use. We are actively working on technology transfer and support for the lead plants.

Thank you for your time.

CHAIRMAN SVINICKI: Thank you. Jason, please proceed.

MR. REMER: Chairman, Commissioners, thank you. It is a pleasure to be with you this morning. Thanks for the opportunity to share an update on second license renewal or subsequent license renewal. Next slide.

What we are going to talk about today is basically how SLR is a bridge to the future; how key safety principles are maintained; how we have optimized the process through this last three years; and what the SLR looks like moving forward. Next slide.

As you know, nuclear plants, our 99 nuclear plants are critical infrastructure. They are the largest and best run fleet of nuclear plants in the world. Everybody uses our technologies. The NRC's rules and guidelines are copied regularly and just pasted into nation's requirements. We have to stay in that leadership position.

As you know, we generate around 20 percent of the electricity, over 62 percent emissions-free electricity, 24/7 always on, fuel on-site. No other source of power in our country can provide what nuclear power provides. In addition to millions and millions of dollars for the local economies, it runs entire counties. Where I was from in Russellville, you either worked at the nuclear plant, you knew somebody that worked in the nuclear plant, or your family was working at the nuclear plant. It supported the economy in those small towns and it continues to across America. We are seeing some of the

shutdown plants, the devastating effect that those shutdowns have on those small plants. Next slide, please.

So how does second license renewal fit into the overall horizon for nuclear power? As you see here, we have laid out kind of an initial idea of where we are at today, the current reactors, advanced technology fuel, how that is going to help us, new light water reactors, hopefully we can get them off the ground, SMRs and then advanced reactors. It is clear that second license renewal is a key bridge to the future. We have to keep the technology for our light water reactor fleet alive through technology, suppliers, labor and training. If our current fleet of reactors shuts down and we lose the capabilities to produce these equipments and staffing to run them, we will have to basically start over. We need advanced reactors to come online or SMRs or other non-light water technologies.

So it is clear SLR plays a critical role in the future of nuclear power. Next slide.

So this slide here is from our May 8, 2014 presentation that I made before you, as Sherry was alluding to. I just pulled it back out. I said let's take a look at that. What's happened in the last three years?

Well, the key principles are still the same for license renewal. The current regulatory process is adequate to ensure acceptable levels of safety. And each plant's licensing basis will be maintained during any license renewal period to the same manner and to the same extent. These are our guiding principles for license renewal, for original operation, and for second license renewal and what might come after that.

What we found is second license renewal is not only possible, but it is a solid bridge to the future of nuclear. I remember when I started about

1	five years ago back in NEI looking at second license renewal, when you talk to
2	folks about going out to 80 years, they kind of give you a twisted look, like are
3	you serious. And now it is worked through. We have asked the questions.
4	We have got the answers and it is no longer just a dream. It really is reality.
5	Second license renewal is sound. Next slide, please.
6	So the regulation is based on 10 CFR Part 54. which

So the regulation is based on 10 CFR Part 54, which anticipated further rounds of license renewal. It didn't say you could only go 20 more years. It anticipates further rounds.

The existing processes ensure safe plant operation. 10 CFR 50, Appendix B, our quality processes, corrective action, all those things going on 24 hours a day at every plant right now. Right as we speak, these processes are maintaining safety.

Aging Management Programs, you have heard a little bit about already. They are solid. We've worked them their first round of license renewal. They work. We made them better.

Maintenance Rule, active equipment -- we don't talk much about active equipment but our maintenance rule has proven itself to be not only adequate but helping us improve plant performance on a daily basis to ensure that our design basis are maintained.

When we started these slides in '14, there were 73 plants that have had license renewal. Now there are 86, 13 more. Back then, 27 plants had entered into the period of the extended operation, greater than 40 years. Right now, there is 42, which if you do the math it comes up to around 128 reactor years in the PEO. So there is a lot of information about how these plants are running in the period of extended operation.

Reliable, predictable processes. We are not seeing any

1	effects. We are seeing plant availability, plant efficiencies at all-time highs
2	They are maintaining a very high level of operational and safety performance
3	Next slide, please.

So the regulation is sound and the regulatory guidance is sound. We used the GALL process, Generic Aging Lessons-Learned, to help us run through the first round of license renewal.

Second license renewal, the GALL has been updated. There has been an extreme amount of work for your staff and our industry as well. We have been heavily involved in that. We have had many, many meetings. I think we actually submitted 300 pages of comments, not because it was so wrong but because it was so big. I don't know if you stacked it up. I think the GALL is you know it is getting close to -- it is nuclear math for sure. A lot of stuff in there. A lot of good stuff. A lot of lessons learned. So we are making it better.

We submitted comments. We got the draft. There has been really good cooperation between your staff, DLR in working on the GALL, as it should be. We are going to have the final copy -- we have had the final copy for stakeholder review. We anticipate the final copy coming out later on in the year.

We also developed a guideline, NEI 17-01, that helps our plants and utilities prepare adequate license packages, SLR packages. That's NEI 17-01. Previously, we have used 95-10. So that is going to be -- that has already been sent to you for evaluation and review.

As well as the safety side, on the environmental side, we are working with your staff to make sure we smooth out any rough edges on the environmental review so that those two processes can proceed together. Next

slide, please.

As you have already heard already, research, our associates at DOE and EPRI have both concluded there are no fatal showstoppers for operation beyond 60 years. Given that you continue to maintain your plants at the high level of safety and performance you are doing and you continue to use Aging Management Programs, you continue to learn lessons. These are living products. They are not static products.

We also -- not only did we -- you know it is one thing to see a PowerPoint slide of the research that is going on but what we did is we said hey, let's go out and see what's going on at DOE, at the National Labs, at some of our key vendors, at EPRI, at some of our licensees. Let's see what they're doing in this area.

So we have hosted seven site visits since 2015. We actually go out in the field. In fact, last week we conducted a site visit at University of Tennessee on concrete. And so that was very helpful. Both the staff and the utilities participated. It has been an extremely valuable process and we have all learned a lot. Next slide.

Quickly, we have optimized the process. Your staff have looked through how we can do better. We have come to agree on an 18-month review schedule which I think is not only doable but we can probably beat that. We have looked at ways to improve efficiencies using electronic communications processes, reducing unnecessary meetings, and being disciplined in our schedule and RAI process on both sides, make sure we write good applications, make sure the staff gets the answers they need. Next slide.

As you are going to hear in a few minutes, our SLR lead plants are on track. It is very exciting. I think they have to go and do a good job in

producing the first applications to get through the process and then I think you are going to see many, many more applications coming through.

We took a survey last year on how many plants might be interested in second license renewal -- next slide -- and this is the data we got back. There is about 20 -- these are sites and so there may be more plants in this but it is 20. It just shows you there is some interest. From what we have heard, everybody that is interested in going forward and can see through this current economic situation I believe would be interested in second license renewal. It is a good deal and it is a good insurance for the future. Next slide.

So in summary, our technical data supports second license renewal. The SLR GALL incorporates lessons learned from the first round. We believe the 18-month schedule is reasonable and doable. Our lead plants are on track. We are going to turn in the highest quality applications possible so that your review can be efficient and productive. And the plant safety is and will be maintained throughout the license period.

Thank you.

CHAIRMAN SVINICKI: Thank you, Mr. Remer.

Mr. Gallagher.

MR. GALLAGHER: Okay, thank you. Thanks for inviting me to talk here today about Exelon's Peach Bottom second license renewal application. And again, I am Mike Gallagher. I have been with Exelon for 36 years. My background is in engineering and operations and I was a senior licensed operator at Limerick Generating Station.

Since 2006, I have been the Vice President of License of Renewal, having responsibility for all the license renewal applications and projects. And over that period of time, we did seven applications for 13 units.

1	I was also the Long-Term Operations Integration Committee
2	chairman in the EPRI Long-Term Operations Committee. That was 2009
3	through 2016.
4	And I am currently a member of the NEI SLR Working Group.
5	Bring up slide 1.
6	So Exelon, we believe Exelon well-positioned to be a lead
7	subsequent license renewal applicant. Exelon and its legacy companies have
8	completed 13 applications for its 22 units. We have 13 units in the period of
9	extended operation right now. And our Clinton facility is the only plant not
10	renewed and we are scheduled to submit that application in 2021.
11	We worked with EPRI and NEI on the initial industry roadmap,
12	which ensured SLR readiness by 2019. And Exelon has heavily participated
13	in the SLR GALL reviews.
14	So based on the fact that the license renewal rule is
15	unchanged, our involvement in the GALL SLR review and its similarity to the LR
16	GALL, and our extensive experience in developing license renewal applications,
17	you know I am very confident that we will submit a high-quality application.
18	We understand the staff is developing an 18-month review
19	plan. We think that is very achievable and we will do our part to submit a high-
20	quality application so that review can be completed.
21	Go to slide 2, please.
22	So Peach Bottom, we select Peach Bottom as our lead BWR
23	plant and we think that is a great plant to be the lead BWR applicant for Peach
24	Bottom was the 8th license renewal application to be approved in 2003 and
25	original license renewal. And we have been operating in the period of

extended operation since 2013.

1	Peach Bottom runs very well. It is highly reliable. We have
2	not had an automatic scram at Peach Bottom in the last 11 years. We have
3	invested a lot into the plant. We have invested over \$1.3 billion in capital
4	improvements in the years 2012 to 2016. We achieved extended power uprate

for both units in 2015 and we get high marks from INPO.

2.1

2.6

At Peach Bottom, we have been part of the community for a very long time. Peach Bottom Unit 1 was the construction started in early 1960s. And so we have had a lot of employees part of that community for a long, long time. We are committed to the community and as an example here, I just put one of our donations have been to local nonprofits. We have donated over \$416,000 last year.

If we go to slide 3. So, our approach to subsequent license renewal is similar to the approach that we had for our license renewal project. And we will be consistent with the guidance in the GEIS and the GALL to the maximum extent possible. The way we do that is you can see here we will do the Part 51 review using the GEIS and it is very -- it has been updated. They are very similar to what has been in the license renewal projects.

And Part 54, again, we have been very involved in reviewing and commenting on the SLR GALL so we understand what's in it and the expectations and it is basically built off of the original GALLs which went up to Rev. 2.

And basically, we will do the integrated plant assessment, the TLA evaluation with the ultimate goal of identifying and establishing the Aging Management Programs.

And so one key point that we want to make sure everyone is aware of is the Aging Management Programs are the key and so that is how we

will manage the aging of the in-scope equipment so that we maintain its intended function. A lot of people think that you know you really have to like predict how long things are going to last. Are they going to last 80 years or this or that. That's really not the case and the rule doesn't require it. Really what you want to do is manage the aging. And if you think about it, that's really the right way to go.

If you had an analysis that predicted the end of life, you'd still have to monitor it and keep an eye on it all along. So really what we want to do is have effective Aging Management Programs so we can monitor the effects of aging and take appropriate corrective action before there is a loss of intended function.

If you go to the next slide, slide 4, this is just the overall process review. You probably have seen this. This is the integrated plant assessment and basically we identified the scope. We identified all the aging effects and materials. And then we do a matchup for all those components to an Aging Management Program and then ensure we have all the Aging Management Programs address the in-scope equipment.

Part of this review, you know we do extensive reviews and we are bringing Peach Bottom's application. We are not like just going to send in the original Peach Bottom application with a few addendums. No, we are starting over and we basically do an update to today's standards. It will look more like our LaSalle application, which was our last application, than our Calvert Cliffs application, which was our first application.

And in this process, we review all the performance at the plant and our internal operating experience. And just to give you an idea of the extent of reviews, we have -- our operating experience is going to put Peach

Bottom, so this is basically corrective actions, it is industry operating experience and so on, we have 365,211 items that we are going through to review. Right now, we are about 85 percent done on that and that is to determine -- make sure we have identified any plant-specific aging effects and plant-specific operating experience for our Aging Management Programs.

And if you go to the next slide, slide 5, it basically just summarizes the programs. Again, we're not done right now. We're still in process but we are anticipating we will have about 48 Aging Management Programs. These all line up with the GALL, the SLR GALL. So this is just the listing of it and it is quite extensive to ensure that, again, we maintain the intended functions of the in-scope equipment.

So in summary on slide 6, you know the GALL, SLR, and the GEIS guidance is comprehensive. We believe it is very clear and has been developed based on learnings from the first renewal research and operating experience and we have been involved in that and reviews of that so we understand it and it has been built off of the first renewal and we understand how to do that.

The Peach Bottom SLR application will be consistent to the GALL, SLR, and the GEIS to the greatest extent possible. We will submit a high-quality application that can support an 18-month staff review and we are on track to submit in the third guarter of 2018.

CHAIRMAN SVINICKI: Thank you very much.

Mr. Aitken, did I pronounce that correctly?

MR. AITKEN: Thank you.

CHAIRMAN SVINICKI: Thank you. Please proceed.

MR. AITKEN: Thank you. Good morning. My name is Paul

1	Aitken and I am the manager responsible for the subsequent license renewal
2	project at Surry. I appreciate the opportunity to come and speak to you today
3	about the progress of Dominion's efforts to prepare for the submittal of the
4	application.
5	So what I will do is I will present a brief overview of Surry Power
6	Station, Dominion's industry involvement in preparing for subsequent license
7	renewal and our current schedule for submittal of our application.
8	So by way of my background, I was involved with all the first
9	renewals for Dominion, starting with North Anna, Surry, working towards
10	Millstone, and then Kewaunee.
11	So with that, next slide, please.
12	Surry Power Station is a Westinghouse 3-loop pressurized
13	water reactor with an output capacity of nearly 1700 megawatts. Together,
14	these two units are capable of producing 15 percent of Virginia's electricity
15	needs.
16	Surry is located in Surry County, Virginia on the south side of
17	the James River, approximately 25 miles upstream where the river enters the
18	Chesapeake Bay.
19	Unit 1 was originally licensed in 1972, while Unit 2 was licensed
20	in 1973. Surry entered the first period of extended operation in 2012 for Unit
21	and 2013 for Unit 2.
22	Similarly, as Exelon, Dominion has invested nearly \$1 billion in
23	capital investments into Surry since the first renewal was issued in 2003. And
24	plans and will continue to invest in Surry to maintain plant reliability for the

current and subsequent period of operation. Next slide, please.

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So I just want to spend a moment and discuss the experience

of the Dominion team. My project team is composed, in part, with individuals involved with the first Dominion renewals like myself. I also brought others from within the industry who had extensive and current license renewal experience to complement the team.

Since the team formed in late 2015, my focus was to become integrated with various industry groups sponsored through NEI and EPRI. As a result of our integration into the various industry initiatives, it has not only been beneficial for Dominion but also for the betterment of the industry. As Mike said, I am currently the Vice Chairman of the EPRI Long-Term Operation Integration Committee, as well as point man for second license renewal or subsequent license renewal within NEI.

So as Mike said, Dominion was directly involved with the NRC staff during the public meetings, heavily involved. The meetings were extremely beneficial to the industry and the other stakeholders to better understand the background on the evolving requirements. We worked through the industry questions and concerns and we were able to find common ground with the staff. I thought that was very beneficial.

So a key takeaway on how we are able to get this point in the process continues to be the open communication that we have demonstrated over the last couple of years. Next slide, please.

I'm sorry. Okay, could we go back two slides where I have data review? I'm sorry, I skipped a slide by accident.

So the decision making to operate Surry for an additional 20 years was founded on the station's very good operational performance. The investments made in the station as Surry being the first station in the Dominion fleet to reach 60 years. It also made sense for the stakeholders and customers,

since Surry Station is one of the lowest cost producers of electricity in the United
States

Dominion employs around 900 employees at Surry, coupled with the tax revenues into the local communities makes operating 20 additional years very significant.

I would also add that pursuing a subsequent license renewal is viewed positively by the State Corporation Commission as a viable energy source for the Commonwealth in the foreseeable future.

As a result, Dominion notified the NRC in late 2015 of its intent to send in subsequent license renewal application in the first quarter of 2019. Based on this announcement, Surry has been identified as one of the lead plants along with Peach Bottom.

Okay, next slide -- next two slides, I guess. I think we covered that, right? There we go. I'm sorry for the confusion.

So I just want to take a quick look at the levels of support needed to develop a renewal application. Developing an application is the responsibility of each utility but there is also a great deal of collaboration and support required with other entities in the industry. To put this in perspective, I want to provide a quick visual representation of the involvement by many groups and companies currently involved with Surry. Next slide please.

Here is what I categorize as the first layer of support. It represents the organizations that you have already heard from today. First, we have the independent research organizations working to address the critical research activities and programs needed for subsequent license renewal. The other support identified reflects utilities like Exelon that we work closely with to provide insights, and we collaborate on industry initiatives, and we peer review

each other's applications.

Lastly, we rely on the leadership at NEI to provide strategic advice, along with facilitation with the License Renewal Executive Working Group, License Renewal Task Force, and the various discipline groups.

NEI stands with the industry and ensures there is an alignment on issues as we work together on second license renewal. Next slide.

And the last circle reflects the various vendors that are currently supporting Surry. Collectively, the diverse group of supporting organizations will provide Dominion the expertise and support to develop a high-quality application, as we will discuss in the next slide.

I believe that subsequent license renewal is really a continuum from the first renewals. The renewal process is stable. It is predictable. And as the regulatory framework remains unchanged, I further believe the renewal of those 86 licenses, as Jason has mentioned, is a testament to the viability of the process.

As the lead plant, similar to Exelon, Dominion will provide an application that incorporates the latest GALL requirements and most recent industry guidance. We will also work closely with our utility and industry partners through peer reviews to draw on the collective experience base to develop a high-quality application.

We also expect to maintain a high degree of consistency with the GALL, to realize the highest degree of efficiency and expected savings in NRC review time. We will demonstrate how the Aging Management Programs will provide reasonable assurance in effectively managing the effects of aging.

Based on this approach, the NRC should be in a position to support an 18-month review.

Let's take a high-level look at the AMPs that we proposed to use. This slide reflects the various Aging Management Programs that will likely be used to manage the effects of aging. My purpose in showing this isn't really to give you an eye test but it is really just to provide a sense of how Surry will align with the GALL.

Just to provide a quick perspective, a fair amount of these Aging Management Programs are currently in place from the first renewals and will require some updating to meet the new requirements but the impact should not be overly significant.

The remaining AMPs will require some changes or enhancements and/or are considered new or one-time inspections that will need to be created and in place prior to entering the subsequent period of operation. I expect I have very good alignment with the GALL. Next slide, please.

So in 2015, NEI published an SLR roadmap to provide information for the public and other stakeholders. The document provided a comprehensive outline on the various initiatives and items that were identified to meet the objective of submitting the lead plant applications, which are shown in the middle there around the 2018-2019 time frame.

I don't know if you have seen this document. I have just a prop here but it is very informative and I would encourage you to take some time to read it, if you haven't.

The time line shown on the slide is within the roadmap document and the time line provides a high-level illustration of some of the markers along the way to support the lead plant applications. It starts in 2009, when the first plant entered the period of extended operation and continues until the first plant reaches 60 years of operation in 2029. Everything in-between is

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1	the work that has been completed or is ongoing to support the industry in
2	preparing for the lead plant applications.
3	As you can see, I bubbled our announcement in late 2015, as
4	well as our intended submittal date of the first quarter of 2019.  And we remain

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slide, please.

So as I wrap up, I first want to commend the NRC staff on their efforts over the last couple of years. The staff has worked very hard in developing the SLR GALL and SRP and provided ample opportunity for stakeholder feedback. We have a great starting point for the lead plants, as well as those likely to follow.

on target with the industry time line with a planned submittal in early '19. Next

I want to reiterate that Dominion has been engaged and integrated with the work leading up to the GALL SLR issuance. We have been heavily invested, along with the others in the industry over the last couple of years, to ensure that we have the appropriate guidance and have explored areas for optimization with the staff based on the vast experiences during the first renewals.

Finally, we are ready to develop a high-quality application using a very experienced team that will provide a high degree of consistency with the GALL, which should ultimately support an 18-month NRC review schedule.

That's all I have. Thank you very much.

CHAIRMAN SVINICKI: Thank you.

Mr. Lochbaum, please proceed.

MR. LOCHBAUM: Thank you and good morning. We appreciate this opportunity to share our perspectives with you and other stakeholders. Slide 2, please.

I will focus on these issues today. We have some other issues
but if we can't sell you on these, we're not likely to convince you about lesser
concerns Slide 3 please

As other presenters have done, I recycled this slide directly from my presentation during the May 8, 2014 Commission briefing. Although 35 months have been ripped off the calendar since then, this point remains valid, as do our concerns. Slide 4, please.

Our first concerns the one-time evaluations of severe accident mitigating alternatives. For example, because the Limerick plant evaluated SAMAs during initial licensing, SAMAs were not reevaluated for their license renewal application. Slide 5, please.

One-time SAMA evaluations rely on three tenuous assumptions that safety innovations will not have emerged, that populations will not have changed, and that costs will not have changed. Slide 6, please.

NEI petitioned the NRC to eliminate the need to submit SAMA evaluations with license renewal applications. The NRC denied the petition for reasons including the legal need to consider new and significant information since the original licensing during this major federal action. Slide 7, please.

The many reasons the NRC found for denying NEI's petition to remove SAMAs from the initial license renewal application seemed equally valid for subsequent license renewal. One and done seems both illegal and imprudent public policy. Slide 8, please.

Our second concern involves the bizarre position and line approach to nuclear safety. Ginna and Point Beach are very similar plants in terms of design and age but because the NRC revised its license renewal standards, Point Beach was required to have an Alloy 600 Aging Management

Program while Ginna was not. Slide 9, please.

If NRC would no relicense Point Beach without an Alloy 600 Aging Management Program, 10 CFR 50.100 gives the NRC the authority to require already relicensed Ginna to have this safety justified measure. 50.100 also gives the community around Ginna the right to expect equal protection from the NRC. Slide 10, please.

If safety warrants that measures x, y, and z be done during the period of extended operation, then the public living around all reactors deserve those protections from undue risk. If such safety measures are not justified, then the owners of all reactors deserve protection from undue costs. Slide 11, please.

If I lived around a reactor, I would wish for it to be late in the subsequent license renewal queue in order to obtain protection for my community. If I owned a reactor, I'd wish for it to be early in the subsequent license renewal queue to get through as cheaply as possible.

The NRC must stop being regulatory leprechauns by granting wishes. Slide 12, please.

The NRC's regulations in 50.100 and 50.109, if followed, would require all relicensed reactors to have the same suite of Aging Management Programs. Subsequent license renewal gives the NRC a second chance to get this right. Slide 13, please.

No members of the public attended the final three NRC public meetings in 2012. Two NRC staffers called me to invite me to the November meetings, even if remotely via phone bridge. I declined on grounds that the concerns raised in the May meeting had never been addressed. If my concerns were going continue to be ignored, I would just save myself and the

staff the charade.

I hadn't realized that the staff would address my concerns four and a half years later. I will block out some time in late 2021 to review their take on my concerns today and save some time in 2022, in case they don't get to it as expeditiously next time. Slide 14, please.

Point Beach was not required to have an Alloy 600 Aging Management Program. True, it could have endeavored to convince the NRC staff that not having one at all was just as good. Maybe that gambit would have worked but it is also true that it costs Ginna much less to have neither a program nor an acceptable excuse. Slide 15, please.

I contended earlier that the NRC is cheating somebody. I contend now and formally request that the NRC Inspector General investigate whether the NRC is violating 50.100 or 50.109, both cannot be met the way the NRC is handling license renewals. Slide 16, please.

I have participated in many processes during which public meetings provided two-way communication between the NRC staff and external stakeholders to facilitate progress toward the outcome and I have participated in the license renewal process. Fortunately, that process is atypical but it is unacceptable and must change if the SLR process is to yield a good outcome.

One might have thought that the NRC staff would address public concerns in the policy paper put before the Commission. One would have been wrong. The NRC staff waited until two years after the commission's vote on the policy issues before addressing public concerns. Slide 17, please.

This is the first car that I bought. It's a 1976 Ford Pinto. I was still in college working towards my nuclear engineering degree at the time. This

now antique car is newer than Peach Bottom, Surry, Oconee, Point Beach, Nine
Mile Point Unit 1, and many other reactors operating today. By the way, I no
longer have this car. It's not so much that it aged badly. It was hard to get
eight-track tapes anymore. Slide 18, please.

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We commended the NRC for its knowledge management efforts begun a decade earlier. The Agency's graying staffers were retiring and being replaced by individuals who type with their thumbs on very small smart phones. Slide 19, please.

As millennials replace nuclear plant workers, they must try to answer safety questions using owner's manuals of varying degrees of detail and try to understand regulatory requirements that may provide scant insights into their bases. Slide 20, please.

Considerable industry and staff resources have been spent in recent years trying to determine whether what was done then needs to be redone now. It is a reflection on a game and not its players. This game is really hard to play, really, really hard. Slide 21, please.

And time won't make this game any easier to play unless steps are taken within the SLR process or in parallel to it. Nuclear plant workers and NRC staffers must be given a proper foundation for making safety decisions in the future.

Thank you.

CHAIRMAN SVINICKI: Thank you all very much for your presentations. We will begin the question and answer period this morning with questions of Commissioner Baran.

COMMISSIONER BARAN: Thank you. Well, thank you all for being here and for your presentations.

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1	There seems to be broad agreement on the key outstanding
2	technical issues that need to be resolved for subsequent license renewal.
3	Richard, your presentation covered DOE's main research
4	areas and schedules and it looks like most of the testing and predictive model
5	development is scheduled to be completed by 2019-2020. How confident are
6	you that the various projects will be completed in those time frames?
7	MR. REISTER: I think we are making good progress on the
8	development of those predictive models and so I am hopeful that we will be
9	successful.
LO	And those models aren't really, just to be clear, I don't feel those
L1	models are necessary for a plant to submit a subsequent license application.
L2	What they are most useful for is predicting long-term degradation so you can
L3	anticipate many years down the road what the degradation might be so there's
L4	no surprises is the main point. And that is really more of an economic issue
L5	than a licensing issue, in terms of being able to get ahead of the degradation
L6	and anticipate it in terms of the maintenance of your plant.
L7	COMMISSIONER BARAN: And so in your presentation, you
L8	said you are not seeing any technical showstoppers that would prevent
L9	operation out to 80 years. Of course there is all this research going on still.
20	What level of confidence should the Commission have that
21	none of the ongoing research will reveal a significant problem with plants
22	operating for 80 years?
23	MR. REISTER: Well when I say technical showstoppers,
24	mean there are no cliff-edge effects that a large portion of a plant would degrade

to a point where it would be uneconomic to make the repairs.

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So the degradation models and predictions that we see, based

on our research, is that it is a gradual process that is manageable by plants.

And so they can make repairs or replacements as needed to the equipment.

It's passive equipment that you would be monitoring during extended operation.

But you know as we have seen, you know some of those repairs might make it uneconomic to continue to operate a plant at some point. But it is a slow process and a manageable process for the most part. That's what I'm trying to say by no showstoppers.

COMMISSIONER BARAN: Okay. Well, let me ask the rest of the panel the same basic question. How confident are you that the outstanding technical issues can be addressed with good Aging Management and Maintenance Programs? Anyone want to share their thoughts on that?

MS. BERNHOFT: I'll chime in next. You said the key word there, that is having the Aging Management Programs in effect and we talked about the technical basis exists for those now.

The continued research really helps with driving out a lot of the uncertainties on how some of the models work, how the aging works. So think of it. You start with a little more conservative approach. Like when we start at management of Alloy 600 we probably did more inspections than were needed. As we gathered data, as we gathered insights from the research, we were able to better focus the inspections to look at the areas of question and concern. It helps with the safety case and it also helps with managing the economics and the outage durations for the plants as we continue to improve our understandings and our models through the research and through gathering the OE. So gathering the inspection results is also an important element to that.

So, it is reducing the uncertainties. And we are also doing a lot of work to improve the inspection technology so you can get faster inspection

results, better information from your inspections.

### COMMISSIONER BARAN: Any thoughts?

MR. REMER: I think the key is the living Aging Management Programs because we collect OE on every plant, on every event, every situation. And when we start seeing operational experience pile up in a certain area, we look back at our Aging Management Program and ask the question are we monitoring this properly to identify degradation.

You combine that with new technology insights and scientific insights and it gives you an extremely high level of confidence that you can operate your plant safely and identify anything that happens early before it leads to degradation.

MR. LOCHBAUM: I would just complement what Jason said by the NRC's Generic Communications Program. If something does emerge, the Generic Communications Program writes a Generic Letter or bulletin has been shown to step in and address that surprise or that new evolving issue. That's a complement to the industry's internal efforts.

MR. AITKEN: Yes, and I would just add as well I think the GALL provides us what we know today and I think we have a good starting point, as I said in my presentation.

And as new information becomes available, there is vehicles to communicate and impose on licensees. And we talked about Generic Communication. We have ISGs, Interim Staff Guidance documents, we have seen a lot of those. So as the information becomes available out of the research area, then we will evaluate and if we have to adjust, we will adjust. But based on what we know today, I think we are on solid footing.

MR. GALLAGHER: I'll say ditto with Paul but I mean we

manage all active equipment and all passive equipment. And the passive equipment is much easier to manage than all the active equipment and we are on top of all that and the plant reliability reflects it.

COMMISSIONER BARAN: Thanks for all those responses. In terms of -- to kind of follow-on what Jason was saying in terms of having these be living programs, Aging Management Programs, I wanted to kind of turn to and ask folks for their views on where Dave kind of ended his -- or maybe you didn't end your presentation but in your presentation raised this concern about different plants being subject to different revisions of the GALL, based on where they are in timing in their applications.

What do others think about that? Is that a problem? Does it make sense for every plant operating beyond 60 years to be using the most recent version of the GALL, including updates as they go so that everyone is using the latest methodologies? How do you all think about that issue?

MR. GALLAGHER: Well, I can address that. I think you know I personally don't think it's an issue because the Aging Management Programs have two elements to them that you have to have and one is operating experience, the other is corrective action. So when Jason they are living programs, that is true.

You know the example of Ginna not having an Alloy 600 program, Ginna has an Alloy 600 program. It wasn't put in place in the license renewal. It was put in place subsequent with ongoing operating experience.

And I think if you step back and think about it, you wouldn't want to have it any other way. Like the folks that came up with the rule were geniuses because it was built off of where you are now but then had the corrective action and operating experience element to it.

Because if you step back and say you want to have a static
program, you want to have everything in there and then do that, first of all, you'll
never get there. And second of all, if that was the case, then once you had
approval in that, you wouldn't be required to make any changes to it. And you
know we learned throughout the years, the research, the operating experience,
and so on. So you want to have that feedback mechanism and I think it is in
there and it is very robust.

COMMISSIONER BARAN: Dave, do you have any further thoughts on hearing Mike's response on that?

MR. LOCHBAUM: If I owned a plant like Ginna and I wanted to save money down the road, I'd just eliminate my Alloy 600 Aging Management Program because I'm not required to have it. The NRC might not like that but the NRC would have no regulatory hook to make me continue that program.

So, again, the whole backfit thing, if it is a justified measure, then it should be required for everybody. If it is not justified, it should be voluntary for everybody. But to have this mishmash of some that is required, some that's not required, makes no sense, legally or morally.

MR. GALLAGHER: Just to respond to that, there is a regulatory hook. You have to maintain the intended function of the in-scope equipment. And so if you don't manage the aging and you have a problem, there is a definite regulatory hook. You know so -- and it is required in your operating experience review program to review that and add it as necessary.

So I think there is a regulatory hook and I think it's there.

That's why the utilities continue to add the programs and manage the aging, in addition to everything else you have to do.

COMMISSIONER BARAN: Let me quickly ask about one of the other issues that Dave raised, which was his concern about not performing updated SAMAs as part of subsequent license renewal. And he made the point well costs change over time, safety innovations over time. What do others on the panel think about that concern or that issue?

MR. GALLAGHER: Well, I can address that. Limerick is a case example. And we did the Limerick application. You have got to step back and look at what the purpose of the SAMA is. The SAMA is part of the NEPA evaluation in the Environmental Report. So it is kind of a mishmash on purpose. If you want to do updated risk analysis, then there should be requirements to do updated risk analysis. And we do do updated risk analysis on -- you know we update the PRAs frequently and that type of thing and any insights are acted upon.

So when you step back and say what was the purpose of the SAMA for the original licensing or license renewal application, it is for the NEPA evaluation.

So then when you do, the rule only requires you to do one SAMA analysis, unless there is new and significant information that would drive you to do another. So in the second renewal, we will do a new and significant review to determine whether or not an additional SAMA is needed to be done and there is a process to do that.

But the success criteria on whether you need to do that is NEPA. And the NEPA is there a seriously different picture in the environmental consequences. And if you step back and think about SAMA, SAMA only if you implement a SAMA, it only improves your risk because you are implementing something that will reduce risk.

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1	So it is really a mishmash of purpose and I think the staff you
2	know the rule was correct in saying okay, let's do it once to figure out if there
3	any significant seriously different it would paint a seriously different picture
4	and in subsequent actions do a new and significant review to see if you have to
5	do that again. So, I think it is well covered.
6	COMMISSIONER BARAN: I am a little over on time but I am
7	curious. Do you have a sense, or Paul if you have a sense, do you have a

curious. Do you have a sense, or Paul if you have a sense, do you have a sense if you reran the SAMAs using kind of current data, do think there would be appreciably different results, or no, or you don't have a sense?

MR. AITKEN: That's a good question. I think what we're finding is we are evaluating the previous SAMAs that we had. We are also looking at all the SAMAs that other utilities have done and we are running cases on -- we started with maybe 150 to 170 and maybe we have 25 and we are going to go through the process of determining if there is new and significant results. And at this point, we don't anticipate any, even with the population or the dose consequences.

COMMISSIONER BARAN: Are you effectively doing an analysis to determine whether any new or significant changes?

MR. AITKEN: Yes.

MR. GALLAGHER: Yes, we do a new and significant review, which is very robust and there is technical reviews in that on other SAMAs. And basically, since the Fukushima mods were put in and the Fukushima FLEX equipment, you know you really don't have a lot of significant cost beneficial SAMAs.

Now, it is not saying you never have a cost beneficial SAMA but usually, and even in the first renewals, if you look at all the SAMAs that were

1	cost beneficial, they are usually very, very small things. You know it is like well
2	if we change these procedures, then or if we put a portable battery someplace
3	and have a procedure on how to hook it up, you can reduce the risk. Well you
4	know we do those kind of reviews when we do the PRA updates. So you know
5	that is I think it is covered like that.
6	COMMISSIONER BARAN: Okay, I'm over my time but Dave
7	are there kind of closing thoughts you had on this since you raised this issue?
8	MR. LOCHBAUM: Well the SAMA, many of those cost
9	beneficial SAMA evaluations were not implemented, like the flood alarm in the
10	switch gear room at Indian Point but they are being litigated. You know those
11	are contentions raised in the Indian Point proceeding. So they must be new
12	They must be significant. They must meet all those criteria because they are
13	still being litigated.
14	The fact that we are going to take that off the table and put it
15	clearly within the purview of the licenses, I don't think that is a sound approach
16	to the issue before the Commission.
17	COMMISSIONER BARAN: Thanks.
18	CHAIRMAN SVINICKI: Thank you.
19	Commissioner Burns.
20	COMMISSIONER BURNS: Thank you. Thank you for the
21	presentations this morning, as we consider this topic or where we are in terms
22	of subsequent license renewal.
23	I had participated in the first round, in the early rounds of license
24	renewal back in the 1990s and I think it is
25	CHAIRMAN SVINICKI: Does that mean you were one of the
26	brilliant people?

1	COMMISSIONER BURNS: Yes, I was looking for that. But
2	also noticed I will be 100 years old if they get their 80 if Peach Bottom or no
3	Surry, if Surry 2 goes to 100 or 80 years of operation, I will be 100. I'm not sure
4	I'll be around.

But anyway, but I think --

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CHAIRMAN SVINICKI: I think with the right Aging Management Program --

COMMISSIONER BURNS: Well I think that's maybe too late for that. I don't know. That's a good point.

But Surry, I think the interesting thing to recall is that there was sort of a move forward and then sort of reconsideration with respect to renewal to get it to where it is today in terms of the approach actually taken.

One question I found the discussion I think very interesting in terms of the differences in GALL reports. And one thing I would reflect on, I might take a different viewpoint in terms of where Mr. Lochbaum does with respect to the legality of differences. I think I would probably take a very different viewpoint. I mean if I actually, if I look back and I can remember doing research over the years in terms of initial regulations. I would love for most regulations to be like they were in the 1950s or 1960s. You just had the darn regulation there and I didn't have to wade through a thousand pages of text like say I have to do on the Fitness for Duty Rule or something like that.

But all kidding aside, I think it is an interesting issue in terms of -- and have had it dealt with as a line attorney in terms of enforcement of license conditions, procedures, things like that. So one of my questions, Mr. Gallagher when you said there is an enforcement hook, the thing that came to mind, is that hook the maintenance rule? Is that what you were speaking of? I'm trying to

1	get more information where you see that nook.
2	MR. GALLAGHER: Well you have to maintain the intended
3	function of the equipment.
4	COMMISSIONER BURNS: Right.
5	MR. GALLAGHER: So if your equipment was inoperable then
6	there would be some type of reportability issue, some type of regulatory issue
7	associated with it.
8	If it was not maintained operable and you didn't take corrective
9	action, you could go through the corrective action Appendix B issues, those kind
10	of things.
11	COMMISSIONER BURNS: Okay.
12	MR. GALLAGHER: So in a broad sense, we have to maintain
13	the equipment that is in scope and we have to maintain its intended function.
14	COMMISSIONER BURNS: Yes.
15	MR. LOCHBAUM: Could?
16	COMMISSIONER BURNS: Sure. Yes, yes. It is an
17	interesting discussion.
18	MR. LOCHBAUM: You wouldn't need a rule at all, you have
19	Appendix B that says you are required to maintain equipment. So why do you
20	have all this nonsense if Appendix B I think the reason for all that nonsense
21	that the geniuses came up with is that it avoids waiting for something to break
22	before you step in and fix it under the regulatory hook. It would be nice if you
23	didn't challenged highly reliable safety systems. Building a wall around
24	Fukushima is a little bit too late today.
25	So I think Ginna is not protected to the same extent that Point
26	Beach is, unless something breaks, and then you have a regulatory hook. I

don't think that's the way NRC wants to regulate safety in the United States.

COMMISSIONER BURNS: Okay one of the other aspects of that discussion that I want to delve into a little bit is the challenge of sort of what I will call older FSARs, although when I say older FSARs, we have had provisions that you have to continue, you have to do continual updates of FSARs so you have a USAR, I guess, and continue that.

But I would be interested in both from Exelon or from Dominion how do you deal with that issue in terms of the plants that you have, that an earlier plant is probably going to have an FSAR that doesn't look like let's say Watts Bar 1 or 2 FSAR does in terms of perhaps the granularity or the depth or that. Are there particular -- and I am talking, obviously, at a high level. It has been a while since I probably have delved into one of them. But how do you think about that issue in terms of that longer term knowledge management issue when you have got people, maintenance people, operations people and all going back over the years during the period of extended operation?

MR. GALLAGHER: Well, I mean there's been a number of initiatives over the years. You know we have design basis documents you know DBDs at the sites and that was in the vein of getting all the current licensing basis together.

COMMISSIONER BURNS: Right.

MR. GALLAGHER: You know the UFSAR, as you said, is maintained and in the context of license renewal we have to have our Appendix A Aging Management Program summaries and commitments in the FSAR and that is put in place right after the license is renewed and then it is updated every two years.

So you know I think we are required to maintain the current

1	licensing basis and that is what we do.
2	COMMISSIONER BURNS: Okay, let me
3	MR. LOCHBAUM: Could I just?
4	COMMISSIONER BURNS: Sure.
5	MR. LOCHBAUM: I think the challenge is if I was a plant
6	worker not maintaining the plant because that is pretty clear but if part of that
7	means replacing something because it's worn out, I have to make sure that that
8	replacement part meets the same safety function of the original. And with the
9	documentation light plants, that decision is going to be a little bit harder than it
10	would be at a more recent plant.
11	So I think it is not so much in maintaining the plants the
12	challenge arises it is in if you end up having to replace a component or a
13	structure because it is worn out.
14	COMMISSIONER BURNS: Okay.
15	MR. AITKEN: I would just say that I mean the equipment, if it
16	breaks and it needs to be replaces, there are processes in place. Sometimes
17	we have to reverse engineering things. That's not uncommon but we do
18	maintain our records and we do have information available so we can go back
19	and figure out what is required as part of the licensing basis requirements for
20	that piece of equipment that hypothetically failed.
21	So we're not in a vacuum. We do have processes in place and
22	they have been effective and they are evaluated by the NRC.
23	COMMISSIONER BURNS: Okay, let me turn to Mr. Reister
24	from the Department of Energy. Of the four knowledge gaps that were
25	identified in this DOE-NRC expanded materials degradation assessment, is

there one that is proving most difficult to fill?

1	And to put it another way, is there one that might have a higher
2	uncertainty higher level of uncertainty with respect to the outcome?

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MR. REISTER: I wouldn't characterize it as higher uncertainty but I think in terms of -- and it is just my personal opinion of what the industry would have to deal with it would be core internals in terms of the need to repair and manage the degradation of core internals. And that could lead to some economic challenges by plants.

The other issues, you know reactor pressure vessel embrittlement is more of a long-term issue. And concrete and cables have pretty good performance. I think they are in the period of extended operation. You know cables will have to be replaced and managed but it is not as high a cost or impact compared to core internals.

COMMISSIONER BURNS: Okay. And Ms. Bernhoft, is there an area that you think is most challenging in terms of the research that is underway for subsequent license renewal?

MS. BERNHOFT: I don't think there is one particular area that is a challenge. I think what is important was touched in is we need to continue to develop the state of the art for the inspection methodology. As the plants continue to age, we do expect things to exhibit aging degradation.

You know you used the example of core internals. You know it is closest to everybody's heart right now but as we improve the ability to do the inspections for the core internals, it gives better information to the utilities for their asset management. And that's making the economic decisions is a repair or replacement needed.

COMMISSIONER BURNS: Okay. And I think a number of you may have mentioned that there is some international cooperation going on

1	in some international work. If you could give me some brief description of what
2	type of work is being done overseas that is contributing to the understanding
3	here, particularly since in terms of outside the U.S., a lot of these countries, I
4	know for example, the French, they are really dealing with first term license
5	renewal. But any research that is worth highlighting.
6	MR. REISTER: Well just as an example, we created an
7	International Committee on Irradiated Concrete. So we collected different
8	countries and France and Japan in particular I think that we are doing work on
9	irradiated concrete and how it was effecting irradiation was effecting the
10	degradation of concrete. So that's just one example where we have an
11	international work.
12	There is some work through the NEA and the IAEA as well.
13	And we also do some bilateral cooperation on various issues.
14	But so we're trying to seek targets of opportunity where there is
15	like-minded research and share that information and not duplicate effort.
16	COMMISSIONER BURNS: Okay, great. Thanks. Oh, go
17	ahead. I'm sorry.
18	MS. BERNHOFT: I can give really just three really quick
19	examples. ERPI is a member of the Material Aging Institute with EDF and
20	TEPCO and some other international members.
21	We do a lot of fundamental work on material characterization.
22	And a project we have going on right there is they built a scale-sized version of
23	a containment concrete or you know the containment model that we'll be doing

some testing on.

Another is working with the Japanese CRIEPI, which is somewhat equivalent to EPRI. We have done quite a bit of work on the

1	modeling for IASCC, a lot of international collaboration on that.
2	And another quick one is we worked with NRC as well and we
3	have harvested materials from the Zorita plant in Spain. It has 36 effective ful
4	power years of operation. It is a small PWR.
5	We have harvested sections of core internals. And we have
6	also just recently worked with them to harvest sections of the biological shield
7	wall.
8	COMMISSIONER BURNS: Okay, thanks very much
9	Chairman, thank you, Chairman.
10	CHAIRMAN SVINICKI: All right. Well, thank you all for you
11	presentations, for your answers to the questions of my colleagues. It has been
12	a very interesting discussion. I might focus on a few different areas, since we
13	have covered a lot of ground already.
14	Jason, your slide 11 had an anticipated has a charge of
15	anticipated submittals per year for subsequent license renewal applications
16	based on a survey that NEI conducted.
17	How would you characterize the dependence of that type of
18	submittals over years? How dependent is that on the reliance on the 18-month
19	review schedule? If the first few of these were to take three or four years, do
20	you think it would paint a seriously different picture of the follow-on applications?
21	MR. REMER: I think that the expectation based on our work
22	with your staff over the last three or four years and the success of first license
23	renewal have given people a confidence to say that it is essentially the same
24	process; we are making some improvements; we have upgraded the AMPs
25	The expectation is that we will be more efficient the second round.

So I think if the first lead plants -- of course we had to get the

lead plants to bite on this idea, right? So the idea that it is going to be better, they were able to take it to their management to get approval to go forward with really a brand new process.

I think if it took a long time that there would be a chilling effect in the industry on second license renewal. I think people would be afraid to possibly submit if there was a lot of delay and it wasn't based on something we did wrong.

So I think it is very important that we stick to the schedule. It is a published 22-month review schedule right now without hearings. And we're looking at 18 months. I think if we can kind of stay within that pocket, we will be fine. If not, there must have been something that came up that we didn't anticipate.

CHAIRMAN SVINICKI: I asked the question, basically, for two reasons. First of all, my understanding of the history is the same as Commissioner Burns. While I appreciate that people have been called geniuses or brilliant or whatever the term was, but the truth was they were hiccups.

COMMISSIONER BURNS: I'm not claiming that title.

CHAIRMAN SVINICKI: Well no, I know you're not because you are a very humble individual but also you are aware of the fact that there were significant hiccups. And the license renewal processes that exist today is an evolution over some hard lessons learned. I think Calvert came in and it didn't go well and there was a fundamental regrouping. And the Agency relooked at the scope of license renewal. And so there were some very foundational changes that were made at that time.

So I can't be the only person in the room who has had the

experience that the first or second time you do something, it is not generally your most efficient. But it may be that the broader perspective is that it is not the first or second time that license renewal itself has been looked at. It might be the first or second or the lead applications for subsequent license renewal but there is a tremendous body of experience that is the foundation. It is the initial license renewal.

So I don't know which way it will go but I kind of ask it for that reason. The second reason I ask it is that we do have a little bit of this foot race. I think all three members of the Commission now will have explored the timeliness of the research results to the initial submittal. So I think we have explored that fully already but the underlying question is are there research results that won't be timely to the initial submittals? And I think we have received answers in the general spirit that that isn't a principle concern right now because the research that is ongoing is going to undergird perhaps some longer term conclusions and the ability to have a better anticipatory knowledge of phenomenology to the relevant and most important Aging Management Programs.

So but it is, I think, a significant, I won't say pressure for the NRC, but it is important that we understand the dependency maybe even for our own planning and resourcing purposes, the dependency in the industry as a whole on the attractiveness of SLR based on keying off that 18-month schedule. There are, of course, separate contested proceedings and adjudicatory processes that, of course, will be available to those who choose to intervene in the process. So that is another element of the license renewal process as we move forward.

1	MR. REMER: Can I say one more thing
2	CHAIRMAN SVINICKI: Sure, Jason, go ahead.
3	MR. REMER: I'm sorry, just to respond is it the reason we are
4	calling them lead plants, rather than pilot plants is that we see it as the first
5	license renewal was a big step and we had to work out the details but second
6	license renewal really follows right along the same path as license renewal.
7	We just made it a little bit better.
8	So that is why they are lead plants and not pilot plants because
9	we don't think there is anything fundamentally different about the first and the
10	second, other than we have upgraded the processes.
11	CHAIRMAN SVINICKI: Well and of course the Commission,
12	itself, in its 2014 decision disapproved the staff's proposal that we undertake
13	rulemaking in this area. And the Commission, at that time, on which I served,
14	validated that the current framework was the right framework for follow-on
15	license reviews license renewal reviews.
16	So that is consistent with the Commission's policy view of the
17	matter.
18	The staff will present next and I don't mean to steal their thunder
19	but they will talk about developments and comments that they received on
20	various documents under development. And they will indicate that the area
21	that received the most comments from industry on the changes that they put
22	forward were to the reactor vessel material surveillance Aging Management
23	Program. And this has to do with the surveillance capsules.
24	So this isn't my area of expertise and I was reading only a
25	summary of this. Is there any concern that there will not be enough of these

surveillance capsules or with these newly instituted -- I was about to call them

1	requirements but of course if it's guidance it can't be requirements but with
2	the new definition of this, are we going to run out of surveillance capsules?

Is there anybody who can address that? I'm not sure what the industry's principle concern was.

MS. BERNHOFT: I will take a stab at that.

With regard to the Surveillance Capsule Program with the PWRs, yes, there were a number of comments on that particular Aging Management Program and I will let the staff better characterize some of those.

Some of them had more to do with like the timing for harvesting of some of the capsules. And we had quite a few comments on that particular aspect that we felt the timing that they were specifying in the AMP may not have given some of the best value for some of the capsules. And it also had a specification that some people have to capture an 80-year capsule, which in some cases has already been harvested.

So the staff was very willing to listen and make some adjustments there. And I think we have ended up in a good place. I'll let Paul speak more to that because he was instrumental in some of those.

But with regard to your question on the number of surveillance capsules, EPRI actually started proactively looking at this in about 2010 and we designed two programs working collectively with the PWR community. One is that we delayed the surveillance -- or the harvesting schedule for 13 capsules so that we could, knowing that plants were going to start talking about 80 years of operation, instead of harvesting them within that 60-year period, we delayed those 13 capsules so that we will collect data out to the 80-year period and they will start to be harvested about 2021 to 2025.

That required license changes or changes in license

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1	commitments. Those letters have been prepared and submitted to the NRC an
2	approved. So that implements that Coordinated Surveillance Capsule
3	Program.
4	In addition, we took some capsules that had already been
5	irradiated and have reintroduced them into two host plants. One has already
6	been reintroduced. The other is going to be reintroduced later this fall. We
7	will harvest those in about the 2025 time frame also.
8	So there will be additional capsule data coming out for the
9	period of operation.
10	CHAIRMAN SVINICKI: Okay. The staff's information
11	indicated that some licensees may have to test and existing capsule that was
12	previously a standby capsule.
13	Again, it's not my area of knowledge but I think there is benefit
14	in having standby capsules because we will be going into further extended

Again, it's not my area of knowledge but I think there is benefit in having standby capsules because we will be going into further extended operations and we may have new phenomenology that we wanted to study. So I think that keeping -- this is very inartful terminology -- but keeping some capsules in reserve for the exploration of emergent issues that we can't see today I think is of value. And it was my hope that we weren't going to use this -- I am probably speaking of this in such amateur terms but we weren't going to use up all the capsules for things right now that we're worried about.

And so it sounds, Sherry, from your answer like we're not in danger of doing that.

MS. BERNHOFT: No, no, we worked hard, knowing this was coming in 2010, to extend out that period for harvesting.

CHAIRMAN SVINICKI: Okay and my last question is one that I am sure that Sherry and Richard don't want me to ask. And it may not make

any sense. So if that is your answer is that my question doesn't make any sense, that's fine.

But so we are working on these models for some of the phenomenology in the materials aging area that we know is relevant or we predict is very relevant to these decisions to operate these plants for 60 to 80, 80 to 100, whatever it is. Your researchers or researchers often look at once I develop a model, maybe it's just me, but the fun thing to do sometimes is to run it way out for long periods of time and see like what it predicts.

Now sometimes what it gives you in the really long periods of time is just junk so it is not really relevant to anything but is there any sense -- do you have any kind of curious researchers that right before they go home at night say let me run this; since the computational availability is a lot cheaper than it used to be, I'm going to run out some of these models and see if I could get a general sense of how long is too long to operate the current generation of operating reactors in the United States. Is that guestion meaningful?

MR. REISTER: Well, I think the models that we are trying to develop are physics-based models, as opposed to empirical models. So, it does have the underlying physics of the chemistry and the thermodynamics and the chemical or interreactions that are physics-based.

So you can run the models. It doesn't take that hard to run them out into time as long as you want to run them. And there are some phenomena that can occur that become significant in the longer periods of operation.

One example, you talked about reactor pressure vessel, there are some concern about late-blooming faces where some of the chemicals, minor materials in the reactor pressure vessel can actually coalesce at periods

of time beyond 80 years kind of in the 80 to 100 years and actually cause some embrittlement that is not seen with just in the current time frames.

And so that is exactly what the models are designed to do is to look out into the future, in the long-term future to see what might happen and try to predict that. And of course part of the modeling would be to take actual materials, either in test reactors that have been accelerated irradiation or harvested form plants and maybe re-irradiated for those longer periods of time to validate the model and say is that something we are actually seeing.

So the physics-based models can actually make those predictions and then you can go back and look and validate it to see is that really going to happen. And you are trying to do that ahead of time to get that 10, 15, 20 years ahead of time prediction to anticipate and eliminate surprises.

CHAIRMAN SVINICKI: The reactors that you're harvesting from like Zion, though, the lifetimes are the lifetimes of the materials and their exposure during the operating life of plants. And I think also there are reactors around the world where materials have been harvested as well.

Did they run long enough to enter into the regimes of the materials' properties where it is terribly good for validating these models?

MS. BERNHOFT: They are good for validation. They're not -- you know you bring up an interesting point. We recently had a workshop on harvesting and one of the comments that was made is that they're good for confirmation but they're not good for prediction.

So remember what harvesting is important for, you know you don't have an 80-year plant out there that we can go take apart. That would be a sweet spot.

But if you want to validate points on your curve, then they can

1	provide a lot of value if it is the right materials and we have a good history of
2	that piece of material, too, as we harvest it. So we spent with Zorita we spent
3	a lot of money on really validating the history of those core internals when we
4	harvested them. And what they showed is that they showed that our models
5	are conservative right now with regard to IASCC.
6	So one other point on Rich's comment, you know they are right
7	they are physics-based models. We validate them with points that we can take
8	from test reactor data or from actual physical data and then we also rely or
9	expert elicitation as well.
10	You know one of the things with the IASCC, it is very much ar
11	international panel. It has been put together and it takes all of those things
12	including the expert elicitation as well to make sure it makes sense.
13	CHAIRMAN SVINICKI: Okay, thank you all. Well, thank you
14	gain for a good discussion.
15	Do either of my colleagues have any additional questions?
16	No, okay.
17	We will take a brief break while we reset for the NRC staff
18	panel. Let's resume at quarter to. Thank you.
19	CHAIRMAN SVINICKI: All right, thank you. We wil
20	reconvene and come back into order here to hear from the NRC staff panel.
21	We will begin with our Executive Director for Operations, Victor
22	McCree.
23	Victor, please lead off the staff's presentation.
24	MR. MCCREE: Good morning, Chairman, Commissioners
25	Baran and Burns. It's good to see you.

During today's briefing, we will provide updates on a number of

items associated with subsequent license renewal.

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As the industry mentioned this morning, we anticipate an influx of subsequent license renewal applications submitted to the NRC in the very near future.

We remain committed to our proactive engagement with both internal and external stakeholders so that our review of subsequent license renewal applications will be as or perhaps more successful than our review of the first license renewals.

Next slide, please?

Today, you will hear from four presenters who will emphasize four key messages.

The first is the license renewal framework is the basis for subsequent license renewal.

Secondly, that the NRC is ready to accept and review high quality sites subsequent license renewal applications.

Third, that we will issue new subsequent license renewal guidance in July of this year following our extensive engagement with stakeholders and a reflection on how best to optimize our reviews.

And, fourthly, confirmatory research is ongoing for a number of the key technical issues that were mentioned this morning related to subsequent license renewal.

Our first presenter to my right is Michele Evans who's the Deputy Director of Reactor Safety Programs and Corporate Support in the Office of Nuclear Reactor Regulation.

Michele will discuss how the established license renewal framework which is contributed to the successful issuance of 87 renewed

1	operating licenses is adequate for the review of subsequent license renewal
2	applications.
3	Our second presenter is George Wilson, to Michele's right, the
4	Director of the Division of License Renewal and NRR.
5	George will provide a status of the development of the
6	subsequent license renewal guidance documents which are expected to be
7	issued in July.
8	George will also discuss how we plan to optimize the
9	subsequent license renewal review process in a way that demonstrates our
10	effectiveness, our efficiency and agility.
11	Our third presenter is Allen Hiser. Allen's the Senior Technical
12	Advisor in the Division of License Renewal and NRR.
13	Allen will provide a high level overview of the changes to the
14	guidance documents as compared to the current revision of the Generic Aging
15	Lessons Learned report, or GALL report.
16	He'll also describe how the subsequent license renewal
17	guidance documents were refined to ensure that the aging management
18	programs and activities will support up to 80 years of plant operation.
19	Our fourth presenter is Brian Thomas, the Director of the
20	Division of Engineering in the Office of Nuclear Regulatory Research.
21	Brian will conclude our presentation with a status of the
22	confirmatory research activities that address significant technical issues related
23	to subsequent license renewal and support up to 80 years of safe operation.
24	So, with that, I'll now turn the presentation over to Michele.
25	Next slide, please?
26	MS. EVANS: Thank you, Vic.

1	Good morning.	Chairman.	Commissioners.

As Vic mentioned during his opening remarks, we are here to discuss how the staff is preparing to receive subsequent license renewal applications.

As you can imagine, many of the activities that were performed to demonstrate our readiness to review a subsequent license renewal application have been derived from our experience with reviewing and processing license renewal applications.

This slide illustrates the overall regulatory process for safety during the license renewal period where the principle change from the initial 40-year operating period is additional aging management of passive long-lived structures and components to ensure safe plant operation during the license renewal period.

#### Next slide?

Okay, the regulations in 10 CFR Part 54 established two fundamental safety principles.

First, the existing regulatory process is adequate to assure safe plant operation for license renewal.

Part 54 establishes additional requirements to address the effects of aging.

Second, each plant's current licensing basis must be maintained throughout the license renewal period.

The regulations for license renewal focused on ensuring that the long-lived passive structures and components in scope for license renewal are aged-managed so that they will continue to perform their intended function during the 20-year period of extended operation.

	01
1	The focus of the renewal review is on the aging management
2	programs associated with passive long-lived components.
3	The application reviews for license renewal include the
4	evaluation of both safety and environmental aspects.
5	NRC's review is accomplished through a combination of on site
6	and in office reviews, audits and inspections.
7	Under a renewed operating license, the licensee is responsible
8	for all existing NRC requirements associated with the current licensing basis as
9	well as additional requirements related to aging management.
10	The reactor oversight process verifies that the licensee's
11	programs and processes maintain plant safety for active and passive system
12	structures and components during the entire operating life of the plant.
13	Next slide, please?
14	In SECY-14-0016, the staff provided potential rulemaking
15	options for subsequent license renewal. In response to that SECY, the
16	Commission did not approve the rulemaking options, leaving in place the license
17	renewal rule that has provided an effective basis for ensuring safe operation
18	during the license renewal period and will continue to be an effective basis for
19	subsequent license renewal.
20	The Commission directed the staff to update its guidance
21	documents for subsequent license renewal and to keep the Commission
22	informed on the progress in resolving technical issues as well as the staff's
23	readiness for accepting a subsequent license renewal application.
24	The staff has provided several briefings and products to the

Commission including a letter to Congress on subsequent license renewal activities sent by NRR in March of 2016, an NRC and Department of Energy

L	Commission briefing in June of 2016 and an Operating Reactors Business Line
2	briefing in July of 2016.

Consistent with the license renewal rule, the focus of subsequent license renewal is on the adequacy of additional aging management programs and activities to ensure safe plant operation during the subsequent period of extended operation.

Our next presenter, George Wilson, will discuss the development of the subsequent license renewal guidance documents.

Next slide?

MR. WILSON: Thank you, Michele.

Good morning, Chairman, Commissioners.

As Michele stated, an important component of the subsequent license renewal application review is determining the adequacy of aging management programs and activities to support operation up to 80 years.

To facilitate a more effective and consistent review, the staff developed a General Aging Lessons Learned for subsequent license renewal report which provides guidance for subsequent license renewal applicants, contains the NRC staff's generic evaluation of acceptable aging management programs and establishes the technical basis for confirming the adequacy for the aging management programs.

The report also provides acceptable methods to manage aging effects up to 80 years of plant operations, although the applicant may propose plant-specific alternatives.

The staff also developed a standard review plan for review of subsequent license renewal applications for nuclear power plants which provides guidance to NRC staff reviewers, ensures quality and uniformity in the

1	NRC staff's reviews and presents a well-defined base from which to evaluate
2	applicant programs and activities for the subsequent period of extended
3	operation.
4	Both subsequent license renewal guidance documents were
5	developed using Revision 2 of the license renewal documents as a starting
6	point.
7	The staff published drafts of subsequent license renewa
8	guidance documents for public comment in December 2015.
9	Dr. Hiser will describe other information that was considered as
10	the staff developed the subsequent license renewal guidance documents.
11	Both documents provide transparency to the public for the
12	subsequent license renewal application review process.
13	Over 300 pages of comments were received and evaluated by
14	expert panels consisting of staff from numerous divisions in NRR and Research
15	The staff also held nine public meetings in 2016 to facilitate
16	discussions of the comments and their resolutions.
17	The staff has completed its disposition of the public comments
18	and revised the documents as appropriate.
19	The staff discussed how the comments were dispositioned as
20	well as the other technical issues related to subsequent license renewa
21	guidance documents during the Subcommittee and Full Committee meetings
22	held with the Advisory Committee on Reactor Safeguards in late March and
23	early April of this year.
24	The final guidance documents will be issued in December of
25	2017 prior to the anticipated receipt of the license renewal applications.

1	As the representatives from the industry indicated, two letters
2	of intent have been sent to the NRC stating that the licensees will submi
3	applications for subsequent license renewal in mid-2018 for Peach Bottom
4	Atomic Power Station and early 2019 for Surry Power Station. However, more

The staff continues to emphasize the usefulness of the letters of intent to inform the NRC of the anticipated subsequent license renewal application submittals so that we can adequately budget and plan for the reviews.

We encourage the industry to continue to submit the letters of intent so we can prepare for the reviews.

# Next slide, please?

applications are expected.

2.3

In the spirit of Project AIM and consistent with the efficiency aspect of our principles of good regulation, a subsequent license renewal optimization working group was developed to evaluate the license renewal application review process and determine areas that could be modified to optimize the overall review process for subsequent license renewal applications.

The optimization working groups obtained feedback from NRC staff during internal meetings and also engaged with the industry and public through several public meetings to better understand their experiences with the NRC staff reviews of license renewal applications as well as their ideas on how efficiencies could be gained.

We are adopting the recommendations of the working group in four key areas.

First, the staff will increase the use of portals to review

1	documents related to the SLR application and the use of teleconferencing and
2	video conferencing for the discussions with the applicant.
3	This will maximize the in-office review time and will help focus
4	the on site audits.
5	Secondly, the staff will fully develop the safety evaluation report
6	and the environmental impact statement prior to attending on site audits to focus
7	the audits on outstanding issues resulting in reduced staffing and time for the
8	on site audits.
9	This is also intended to minimize the potential for additional
10	rounds of Requests for Additional Information and help further focusing on site
11	audits.
12	Thirdly, the staff will eliminate redundant inspections for
13	subsequent license renewal. This will eliminate the scoping and screening
14	activities associated with the preparedness to implement aging management
15	programs which were fully vetted for the license renewal.
16	Lastly, the staff will hold a public meeting to explain the generic
17	issues associated with the required consultations for the environmental reviews
18	for other federal agencies.
19	This will ensure that the licensees are aware of the required
20	biological studies necessary to ensure that the consultations with other federal
21	agencies are effective and timely.
22	A license renewal application review is typically performed in
23	22 to 24 months for a high quality application.
24	We project that we can reduce our safety review time by about
25	25 percent by implementing these efficiencies.
26	The overall review time could be achieved in 18 months after

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1	completing the acceptance review.
2	Of course, this relies on receiving a high quality application with
3	no unusual technical issues or contentions as well as timely responses to any
4	requests for additional information from the licensee.
5	We will continue to look at our processes and schedules as we
6	progress with SLR reviews and apply lessons learned as we go.
7	I will now turn the presentation over to Dr. Allen Hiser. Aller
8	will discuss the staff's process for developing the staff's renewal guidance
9	documents.
10	Next slide, please?
11	MR. HISER: Thank you, George.
12	Good morning. The Commission SRM regarding
13	SECY-14-0016 directed the staff to keep the Commission informed on the
14	progress in resolving several technical issues related to subsequent license
15	renewal, identifying the four issues on this slide related to the reactor pressure
16	vessel, vessel internals, concrete, and electrical cables.
17	You will hear more about these issues in the next few slides
18	and from Brian Thomas later.
19	Overall, the industry is responsible for developing a technica
20	basis to demonstrate that aging effects for these and other technical issues wil
21	be managed for subsequent license renewal.

For those aspects of these technical issues that are not fully resolved on a generic basis, the applications for subsequent license renewal will need to address these issues on a plant-specific basis.

Thomas will describe in a few minutes.

The NRC is carrying out confirmatory research, as Brian

### Next slide, please?

To implement subsequent license renewal, the staff has developed two main guidance documents, the GALL-SLR and the SRP-SLR which parallel the Generic Lessons Learned Report Revision 2 and a Standard Review Plan for License Renewal Revision 2 that are used for license renewal.

The GALL-SLR report identifies those material environment and aging effect combinations that require aging management, during the subsequent period of extended operation, including identification of appropriate aging management programs.

Applicants are responsible for identifying in their applications any additional items that require aging management.

Although the GALL-SLR report identifies one acceptable approach to manage aging effects, applicants may propose plant-specific alternatives along with sufficient justification that the program will adequately manage the aging effects.

While the GALL-SLR report is geared toward use by applicants to identify acceptable aging management approaches, the SRP-SLR provides guidance to the staff for its review of SLR applications.

The SRP-SLR ensures a consistent and transparent review of SLR applications by documenting the acceptance criteria and review procedures that will be used by the staff in its reviews.

# Next slide, please?

In order for plants to operate for 60 to 80 years during their subsequent period of extended operation, we needed to determine the aging effects that could occur during the operating period up to 80 years to ensure that these aging effects can be detected in a timely manner and appropriately

managed.

The relevant aging effects could include known mechanisms of age related degradation that are found in new locations or are found to be more severe than previously identified due to exceeding incubation times or activation energies that govern the mechanism.

Further, new phenomena that could induce aging represents another area of interest. We need to remain alert to new aging related phenomena that can occur.

Because the plants have operated for less than 50 years, other information beyond an exclusive reliance on operating experience is being used to identify the aging concerns for the operating period up to 80 years.

To that end, in 2014, we formed subsequent license renewal expert panels from staff in the Office of Nuclear Reactor Regulation and the Office of Nuclear Regulatory Research to begin developing the guidance documents for subsequent license renewal.

These panels started with the Revision 2 versions of the GALL report and the SRP-LR that were developed for license renewal.

The panels then reviewed and deliberated on the information contained in various sources as outlined on this slide.

We reviewed the information from the expanded materials degradation assessment which Rich Reister described earlier in which NRC, DOE, industry, and international experts identified the areas that they believe will be the most challenging for subsequent license renewal.

In addition, we reviewed reports from audits that were performed at several plants that had operated for several years in a period of extended operation.

1	These audits were intended to qualitatively assess the
2	effectiveness of the AMPS and to identify unexpected aging phenomena.
3	These audits reviewed all of the AMPS implemented at the
4	plants, including those implemented on a one-time basis to ensure effectiveness
5	of preventive programs such as water chemistry programs as well as aging
6	management programs that implemented on a periodic recurring basis.
7	We also reviewed operating experience from both domestic
8	and international plants within these panels.
9	And, lastly, we considered comments from stakeholders that
10	were collected during public meetings as well as comments from the staff.
11	Next slide, please?
12	In the next several slides, I will describe some of the
13	refinements in the GALL-SLR report and the SRP-SLR as compared to the
14	Revision 2 versions of the GALL report and SRP-LR that are used for license
15	renewal from 40 to 60 years.
16	Two new aging management programs are included in the
17	GALL-SLR report. One program addresses fluence monitoring of the reactor
18	pressure vessel and reactor vessel internals as applied to both time limited
19	aging analyses and aging management review for the vessel and internals.
20	This new program provides a consistent generic approach for
21	existing plant programs that are used to monitor neutron fluence.
22	In addition, a new program to manage aging of high voltage
23	insulators is including the GALL-SLR report.
24	The inclusion of this new aging management program provides

25 a generic aging management approach in lieu of the previous treatment on a 26 plant-specific basis by each applicant.

1	This new aging management program is an example of our use
2	of lessons learned from review of prior applications to improve both the
3	applicant's development of its subsequent license renewal application and also
4	the staff's review of the application.

2.6

The aging management program for reactor vessel internals of pressurized water reactors and the Generic Aging Lessons Learned report utilizes as its basis an industry report that addressed conditions for license renewal at 60 years of plant operation.

This program uses a sampling approach to inspect the components that are the leading indicators for degradation.

For example, those with the highest neutron fluence or stress levels.

The industry has stated that it will submit a similar analysis for 80-year conditions in 2020. In the absence of this generic 80-year analysis, GALL-SLR states that each applicant may use its existing program for 60 years as long as they supplement the program with a gap analysis to identify the aging — the additional aging management activities, if any, that would be necessary to ensure adequate aging management for 80 years.

We received the most comments from industry on the changes we drafted to the reactor vessel materials surveillance aging management program.

The updated guidance for this program ensures that plants will develop sufficient vessel material surveillance data to cover the entire subsequent period of extended operation.

Specifically, licensees are expected to test the surveillance capsule that bounds the fluence the vessel would see over 80 years of

operation.

However, we do not want licensees to accomplish this objective by deferring testing of a license renewal capsule that they plan to test to address 60 years of operation.

Many licensees will need to update their surveillance programs for subsequent license renewal, some by testing an existing capsule that was previously a standby capsule and others by reconstituting samples from tested capsules and reinstalling these samples in a new capsule in the vessel.

In this way, we have timely verification that the vessel is operated with appropriate safety margins consistent with NRC regulations and an early indication of any potential concerns.

### Next slide, please?

In the electrical cable area, one program for electrical insulation of inaccessible cables that are underground and not subject to environmental qualification requirements was expanded into three aging management programs to account for aging of cables at low and medium voltages along with instrument and control cables.

This change was made to address the differences in both aging effects and testing for the three types of cables.

For concrete, we have made changes to the guidance to address aging management of alkali-silica reaction and irradiation of concrete, both of which are covered by further evaluations that require plant-specific attention.

For alkali-silica reaction, the further evaluation identified in Revision 2 of the GALL report for license renewal was updated to include recent operating experience and additional understanding of this mechanism.

1	A new further evaluation on irradiation of concrete provides a
2	means for applicants to identify plant-specific conditions that would indicate
3	either a need for aging management or a technical basis for concluding that
4	irradiation of concrete is not a relevant aging issue for the plant.
5	Next slide, please?
6	The guidance documents for subsequent license renewal
7	provide means for applicants to develop adequate programs and for the plant
8	or for the staff to review applications.
9	Most of the relevant aging issues are addressed by generic
10	aging management programs in these documents.
11	For the few remaining issues, the guidance documents identify
12	further evaluations for the applicants to describe and justify their plant-specific
13	proposals to manage the aging effects.
14	In all cases, applicants are responsible to ensure that they have
15	identified the relevant aging issues for their plants, appropriate aging
16	management activities, and adequate justification for their programs.
17	With the completion and issuance of these guidance
18	documents, the NRC is prepared to review subsequent license renewal
19	applications.
20	I will now turn the presentation over to Brian Thomas who will
21	discuss the confirmatory research activities that are occurring related to
22	subsequent license renewal.
23	Next slide, please?
24	MR. THOMAS: Thank you, Allen.
25	Good morning, Chairman and Commissioners.
26	As Dr. Hiser has outlined earlier, I will provide a status of the

1	confirmatory research supporting SLR regulatory decision making focusing on
2	the four key technical issues identified in the 2014-SRM.
3	In that SRM, the staff has made progress since that SRM, the
4	staff has made progress in conducting research to resolve the key technical
5	issues.
6	This progress has been accomplished through a staff research
7	coordination with DOE and EPRI through deep-dive meetings and other
8	meetings on electrical cable aging, concrete degradation, and vessel internals.
9	The progress resulted in enhanced aging management
10	programs which are addressed in the subsequent license renewal guidance
11	documents as Dr. Hiser just discussed.
12	Subsequent license renewal applicants need to address the
13	issues not resolved on a generic basis with plant-specific basis submittals.
14	Data and knowledge from near-term and longer term
15	confirmatory research will be obtained to augment SLR guidance as needed.
16	First, I will address research on the reactor pressure vessel
17	embrittlement.
18	As one aspect of maintaining reactor pressure vessel integrity,
19	the NRC requires licensees to monitor the embrittlement of the reactor vessel
20	steel through surveillance programs. These programs are conducted
21	according to the requirements of Appendix H to 10 CFR Part 50.
22	Using the data from these programs as input to a
23	well-established predictive model, information on the future state of the vessel
24	embrittlement can be obtained.
25	For example, surveillance capsule specimens withdrawn and

tested over 40 years of operation could have experienced radiation exposure

1	equivalent to that of the vessel wall after 60 to 80 years of operation.
2	The graph on the right of the slide shows the U.S. operating
3	experience resulting from these surveillance programs.
4	The staff continues to use these data to assess the accuracy of
5	the embrittlement prediction model in the NRC's Regulatory Guide 1.99
6	Revision 2.
7	Allen Hiser discussed a new aging management program
8	included in the GALL-SLR report that will provide a consistent generic approach
9	for existing plant surveillance programs.
10	The capsules from such programs will be removed from their
11	host reactors in the coming decade.
12	Additionally, EPRI has undertaken supplemental surveillance
13	programs to obtain data at high fluence for the PWR fleet.
14	The staff will use the data from these programs to assess the
15	continued validity of the NRC predictive model and to confirm the adequacy of
16	the RPV aging management programs.
17	Next slide, please?
18	The objective of the staff's confirmatory research on reactor
19	vessel internals and primary system components is to independently assess the
20	effects of irradiation on material properties.
21	Irradiation can lead to irradiation assisted stress corrosion
22	cracking, IASCC, loss of fracture toughness, and joint swelling resulting in
23	potential loss of materials performance and component integrity during the SLR
24	period.
25	Materials removed from decommissioned plants are being

tested through international collaborative programs and at the Argonne National

1	I aboratory and are subjected to additional irradiation at the Holden reactor to								
Τ.	Laboratory and are subjected to additional irradiation at the Holden reactor to								
2	confirm the current understanding of the aging effects.								
3	This additional irradiation will simulate the irradiation assisted								
4	degradation expected in some critical components during the SLR period.								
5	The NRC's collaborating with EPRI and with international								
6	regulators to obtain data on irradiation assisted degradation representatives of								
7	materials during the SLR period.								
8	For example, the Zorita Internals Research Project is								
9	examining irradiated material specimens from the decommissioned Zorita plant								
10	in Spain, which operated for approximately 30 effective full power years.								
11	The data from this program will offer insights on the irradiation								
12	damage expected during the SLR period.								
13	The staff will use the confirmatory research results in their								
14	evaluation of the revision to EPRI's MRP-227 report from the Materials								
15	Reliability Program entitled Pressurized Water Reactor Internals Inspection and								
16	Evaluation Guidelines.								
17	The results from this research will also be used to confirm the								
18	adequacy of inspection plans and other aging management guidelines.								
19	Next slide, please?								
20	Degradation of concrete structures is another technical issue								
21	that the staff is examining. The NRC's cooperating with the National Institute								
22	of Standards and Technology to evaluate the effects of alkali-silica reaction on								
23	nuclear power plant concrete structures through the SLR period.								
24	This project will assess the effects of ASR and structural								

performance of concrete under design basis static and dynamic loads for current operating conditions and performance into the license renewal SLR

2.6

The NRC is participating in an international cooperative research program to assess structures subjected to concrete pathologies under the Nuclear Energy Agency.

In addition, the NRC is involved in a collaborative program with the technical support organizations for the French regulator.

The material characterization and structural performance data from our research will inform the review of the plant-specific evaluations and the development of generic guidance for the review of aging management programs.

## Next slide, please?

The staff is coordinating with the Department of Energy on concrete irradiation damage and with EPRI on structural safety performance of concrete under design basis loads to support confirmatory reviews.

The staff is also assessing EPRI's approach to evaluate which plant structures and configurations are most susceptible to the irradiation damage.

Also, the staff is reviewing models of neutron fluence and gamma dose to determine whether degradation of biological shield concrete is a concern during the SLR period.

Based on the reviews, the staff will determine whether additional confirmatory research on irradiated concrete is warranted. The results will inform any necessary future refinements in SLR guidance.

## Next slide, please?

The staff is also evaluating the adequacy of the electrical cable condition assessment techniques in harsh environment qualification methods.

1	The staff has recently begun confirmatory research on aging of							
2	cables that mimic up to 80 years of operation.							
3	The cable samples harvested from ex-plant and laborator							
4	aged, will be subjected to a loss of coolant accident test to evaluate performance							
5	of their intended safety function during and after a design basis event.							
6	The staff is assessing synergistic or concurrent thermal and							
7	radiation aging processes and evaluating condition assessment methods for							
8	detecting changes in electrical, mechanical, and chemical properties of cables.							
9	The industry's performing research to determine the							
10	susceptibility of low and medium voltage cables to degradation and potential							
11	failure in wet and submerged environments.							
12	The staff is coordinating with EPRI in evaluating the need for							
13	additional confirmatory research on wet and submerged cables.							
14	The NRC is also participating in international cooperative							
15	research on cable assessment under the auspices of the Nuclear Energy							
16	Agency.							
17	This program aims to establish the technical basis for							
18	assessing the qualified life of electrical cables in light of the uncertainties							
19	identified following the initial qualification testing.							
20	In this and related areas, the staff will likely continue research							
21	for the next three to five years to confirm the adequacy of the safety basis for							
22	SLR and refinement of the aging management programs.							
23	This concludes my presentation and I would like to turn over							
24	the presentation to Victor McCree.							
25	MR. MCCREE: Thanks, Brian.							
26	Chairman and Commissioners, we appreciate the opportunity							

1	to provide you an overview of our proactive activities to prepare for subsequent							
2	license renewal.							
3	In light of these activities, we feel that we're well positioned to							
4	review subsequent license renewal applications that we'll soon receive.							
5	Furthermore, we have confidence that the framework used for							
6	license renewal remains sound and is sufficient to facilitate successful							
7	subsequent license renewal application reviews.							
8	Throughout this process, we've engaged representatives from							
9	the industry on the guidance that will be issued in July so that they can get an							
10	early start in preparing their applications.							
11	We've also captured ideas and recommendations from external							
12	stakeholders and used them to optimize our safety review process so that we							
13	can be more efficient and effective.							
14	We value those exchanges which are ongoing and we'll							
15	continue to engage in them so that our subsequent license renewal activity							
16	efforts are optimized.							
17	As you heard, there are confirmatory research efforts underway							
18	on the technical issues that were described today.							
19	If any of this changes the basis for our generic guidance, we'll							
20	leverage our current well-functioning guidance update process to make							
21	appropriate changes including engaging in public communications on such							
22	issues.							
23	So, with that, we welcome any questions.							
24	Thank you.							
25	CHAIRMAN SVINICKI: Thank you to each of the staff							
26	presenters for that information.							

1	We will begin, once again, with Commissioner Baran.
2	COMMISSIONER BARAN: Thanks.
3	Well, thank you for your presentations and for all of your work
4	in this area.
5	We talked a bit about reactor vessel internals during the first
6	panel, baffle bolt degradation is an example of an aging management issue
7	related to internals.
8	I think one of Sherry's slides actually had a few baffle bolt
9	pictures on it.
10	The bolts degrade over time due to radiation exposure and
11	differential pressure stresses in the core.
12	Recently, licensees have discovered a surprisingly large
13	number of degrade baffle bolts at four loop PWRs with a down-flow
14	configuration.
15	Although the NRC staff was aware of this degradation
16	phenomenon, the number of bolts with indications of problems was much higher
17	than expected.
18	For example, visual and ultrasonic testing revealed that around
19	a quarter of the baffle bolts at Indian Point Unit 2 showed indications of
20	degradation.
21	Based on Unit 3's shorter run time, the staff predicted fewer
22	degraded baffle bolt indications at Unit 3. But, again, the testing results were
23	surprising. Unit 3 turned out to have more indications of degraded baffle bolts
24	than the older Unit 2.
25	What's the staff doing to understand the gap between what was
2.6	expected and what actually occurred?

1	MR. MCCREE: If we'd have Jeff Poehler							
2	COMMISSIONER BARAN: Sure.							
3	MR. MCCREE: from the staff take that question.							
4	MR. POEHLER: I'm Jeff Poehler, Senior Materials Engineer							
5	in the Vessel and Internals Integrity Branch in NRR, Division of Engineering.							
6	So, yes, I just wanted to say, yes, at Indian Point Unit 3 there							
7	was a larger number of baffle bolts found potentially degraded than at India							
8	Point Unit 2 by about it was about 30 bolts more, 259 potentially degraded							
9	bolts versus 227 which is, you know, is a little bit counter to what was predicted							
10	based on the operating history of the plants.							
11	But, when we looked at the degradation, the actual severity of							
12	the degradation at Indian Point 3, although the numbers of bolts were somewhat							
13	greater, it appears that the degradation of the bolts is actually less severe.							
14	And, if you look at based on removing bolts, when you remove							
15	the bolts with indications, if they break, that's an indication of a more advanced							
16	state of degradation.							
17	So, at Indian Point Unit 2, about 30 percent of the bolts broke							
18	upon removal. Whereas, only about 15 percent of the defective bolts or							
19	potentially degraded bolts are breaking at Indian Point 3.							
20	So, that's one indicator that the indications may be smaller, in							
21	other words, the cracks in the bolts are smaller and not as old or not as							
22	advanced.							
23	Also, at Unit 3, there was no bolts that were observed to be							
24	visually failed. While, at Indian Point Unit 2, there are about 30 bolts that were							
25	visually found completely broken before they even began the UT examination.							
26	So, based on those indicators, although the numbers appear to							

1	be larger at Unit 3, we don't necessarily think it's a worse state degradation.
2	And, in order to help us understand this better, the staff is
3	keeping close tabs on some destructive laboratory work that the industry is
4	doing. They've withdrawn quite a few bolts from at least two of the first three
5	plants last year that found large numbers of bolts degraded.
6	So, those laboratory exams are ongoing. We've seen some of
7	the preliminary results and we're following that closely.
8	And, we're also following closely what the industry is doing as
9	far as developing revising their guidance. And, we've already received two
10	interim guidance letters issued by the EPRI Materials Reliability Program that
11	the staff is currently evaluating.
12	We held a public meeting two weeks ago to discuss the latest
13	guidance letter.
14	So, we will eventually document our position on that in a
15	publically available document.
16	COMMISSIONER BARAN: Let me ask about the guidance.
17	Are you talking about surveillance requirements guidance?
18	MR. POEHLER: This is guidance that relates to the inspection
19	schedule for both the initial inspections of the baffle formal bolts and subsequent
20	inspection.
21	Once they do the initial inspection, how long do they have to do
22	the next inspection, the follow up inspection?
23	MR. HISER: This guidance impacts directly on the aging
24	management program.
25	COMMISSIONER BARAN: Right.
26	MR. HISER: So, the PWRs have implemented and that are

1	addressed in the renewed licenses and will be addressed as well in the
2	subsequent license renewal.
3	COMMISSIONER BARAN: Okay. And, what's the
4	sense so, it sounds like that effort's ongoing in terms of updating the
5	inspection guidance.
6	What is the sense about the adequacy of the current inspection
7	schedules, timing and nature of inspection? Is it the view that there are some
8	changes that should be made to the inspections?
9	MR. POEHLER: Yes, well, the staff agrees that, for a certain
10	category of plants, there's seven plants that were in the most susceptible
11	category and those are the plants Westinghouse units with four loops and a
12	down-flow configuration and Type 347 stainless steel bolt material.
13	So, that includes the Indian Point units, DC Cook, Salem, and
14	one of the Diablo units.
15	So, it appears that the initial schedule that was originally in the
16	MRP guidance, MRP-227, was a little bit maybe optimistic for those plants.
17	But, for all the other plants, all the other PWRs, it appears to be
18	right on. We have quite a few other Westinghouse units like two and three loop
19	downflow units that are older units and they seem the guidance seems to be
20	appropriate for them, at least as far as the initial inspection schedule.
21	Although, even the latest guidelines from EPRI have moved
22	that up a little bit. Before, they had until 35 effective full power years, now they
23	have to move back to 30.
24	And, some of the other categories of plants are still allowed to
25	go out to 35 EFPY.
26	COMMISSIONER BARAN: So, for the seven plants you're

1	talking about there, what you're envisioning, I understand that decisions aren't
2	made on this, what you're envisioning is that initial inspections would start earlier
3	on this and that there may be increased frequency of those inspections?
4	MR. POEHLER: Well, yes. They've already started earlier.
5	In fact, the first interim guidance came out last summer. So, four of those
6	seven plants have already implemented their done their baseline initial UT
7	examinations and the other three will complete them by the end of this year.
8	One of them is doing it this week and another starts next week
9	and then one in the fall.
10	So, they'll all have done their initial exams. And then, their
11	follow up exams, they've also the latest interim guidance from EPRI MRP also
12	puts some limits on how many years they can go before they have to do a follow
13	up.
14	And, they also have to do a plant-specific evaluation to verify,
15	you know, how long they can go. It might be less, it might it could be only
16	two years and some plants, like Indian Point has not voluntarily, but they've
17	modified their program to do the UT at each subsequent refueling outage until
18	they shutdown.
19	COMMISSIONER BARAN: And, what's the time line for the
20	staff deciding on guidance?
21	MR. POEHLER: As far as reviewing the industry's guidance,
22	we're doing that right now. I don't know if I have an exact date for when we'll
23	be publish our final position on that, but I think it will be this year.
24	We also, in house, we are also reviewing a revision to the
25	MRP-227 guidelines. That's the overall guidelines for all the internals, not just

baffle bolts. And so, that is underway and the safety evaluation for that would

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But, in that, we would also document our position on the EPRI interim guidance for baffle-former bolts because the interim guidance actually modifies the standard guidance.

COMMISSIONER BARAN: Thank you. Thank you for that update.

Baffle bolts, I think, raise the broader question of how NRC's going to address unexpected new technical issues or results in the context of subsequent license renewal.

We know there are outstanding technical questions related to operation beyond 60 years, but, as Allen mentioned, new issues are also likely to arise that we aren't currently focused on.

How are we going to address those issues in the subsequent license renewal process or, put another way, how do we ensure that aging management programs can adequately address the unknown unknowns?

MR. WILSON: A couple of the ways that we do that is, as we get new operating experience and we evaluate new issues, the staff issues interim staff guidance so that industry understands that they have to address the issues.

We also, if we would have the applications in house, we would ask RAIs on those issues to the applications that we had in house because the ISGs would be forward looking.

We also, one of the things the industry pointed out that is correct, the aging management programs are living. So, we actually have inspection processes that will go back out and look five to ten years in the license renewal.

		١	Ve ju	ust m	ade a	brand nev	v ins	pection	, 7103 Phase	IV, which
will	actually	go	out	and	that	evaluates	the	aging	management	program
effe	ctiveness	and	d we	evalu	iate th	nat.				

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It's actually a performance-based inspection review. It starts out with 7103 Phase II which does a baseline inspection and looks at the performance of the licensee.

Then the regional management along with headquarters management decides if, like if it was a dual unit site, whether we would go do another full blown team inspection in the second unit or the first unit did fine and we would just send a smaller team out and only focus on those areas.

If we see a degrading performance, then there's another Phase III inspection that we could do at the one to two year point that they were in license renewal or we could pull that based on the performance, we would work with the region.

So, those are some of the ways that we look at the overall effectiveness of the aging management programs for each individual plant. And, we would expect them to -- everything's in their FSARs and their Corrective Action Programs and they would make adjustments as necessary because they have to keep the aging management programs effective to, you know, prevent long-term degradation and make sure that safety systems can perform their function.

COMMISSIONER BARAN: And, I'm going to try not to go as over as I did with the first panel, so let me just close with this.

So, you know, Dave Lochbaum on the first panel raised this point about, you know, as time passes and there's more operating experience and lessons learned, we're seeing revisions to the GALL reports and to the

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1	methodologies and the AMPS.
2	And, he argued that, well, it's a problem that, you know, later
3	plants are basically subjected to or complying with better versions of the
4	guidance than earlier plants.
5	Do you have thoughts about that?
6	MR. WILSON: No, I think they, once again, it's living. And, if
7	you look at the way that a license renewal rule Part 54 was written, let's say like,
8	you know, he brought the alloy 600, if it becomes, after even after we issue a
9	license, a renewed license to a licensee, if there's an SSC that should have
10	been scoped and that we originally didn't scope, then the licensee should look
11	at that, evaluate that, put their Corrective Action Program, and either modify an
12	aging management program or a TLA they have at the site or make a new one
13	which is in accordance with 54.37.
14	So, the regulation has that along with the Corrective Action
15	Program, we'll expect the licensee to make those necessary adjustments as
16	new operating experience comes throughout the plan.
17	COMMISSIONER BARAN: Thank you.
18	CHAIRMAN SVINICKI: Thank you.
19	Commissioner Burns?
20	COMMISSIONER BURNS: And, let me follow up give a
21	follow up question to that.
22	So, what you're expecting to do is for the licensee to continue

So, what is the consequence of the failure to do that? Is that a violation? I don't recall the --

to evaluate the aging management program to make adjustments to reflect

operating experience, industry-wide experience.

23

24

25

1	MR. WILSON: I would have to
2	COMMISSIONER BURNS: provision in Part 54.
3	MR. WILSON: Well, a lot of this falls back into Appendix B
4	Criterion 16, the problem identification and resolution to the Corrective Action
5	Program that a licensee has.
6	As additional information comes in, we would do an expectation
7	that the licensee evaluate to see if they have to do any necessary adjustments
8	If we did not agree with the licensee and they continue not to
9	make, then we could take enforcement actions under 50.100 and/or 50.109 the
10	backfit or, you know, the orders.
11	But, right now
12	COMMISSIONER BURNS: On the basis of Appendix B or
13	MR. WILSON: Correct.
14	COMMISSIONER BURNS: Okay.
15	MR. WILSON: Because
16	COMMISSIONER BURNS: Because the violation isn'
17	50.109.
18	MR. WILSON: No, well, that would be looking at doing a
19	backfit.
20	COMMISSIONER BURNS: Right, okay.
21	MR. WILSON: So, you were asking what would I do with
22	enforcement, I was just running through
23	COMMISSIONER BURNS: Okay.
24	MR. WILSON: the different scenarios. But, we would first
25	start out hope that the licensees their Corrective Action Programs, we evaluate
2.6	then they make the necessary adjustments in accordance with their programs.

1	COMMISSIONER BURNS: Okay.
2	MR. HISER: One of the difficulties
3	COMMISSIONER BURNS: Go ahead.
4	MR. HISER: One of the difficulties with a lot of the things that
5	Mr. Lochbaum mentioned is that aging management is intended to preserve
6	margin. So, it's to maintain margin against having a safety issue.
7	50.109 puts a very high hurdle on the staff to implement a
8	backfit. You really have to have a safety issue.
9	So, if now you have just a slight degradation in the margin,
10	you're not at the level that you can implement a backfit because of 50.109.
11	COMMISSIONER BURNS: Okay. Let me address one other
12	thing and give the staff an opportunity to address.
13	Mr. Lochbaum was, I think, probably, as I would expect and
14	probably would be disappointed that it took four years for the staff to respond to
15	a comment he made. How would the staff respond to that?
16	What is their rationale for that or an explanation?
17	MR. BLOOM: Steve Bloom, the Branch Chief in charge of
18	Generic Subsequent License Renewal Operations and Guidance.
19	I don't want to throw anyone under the bus, but before I got to
20	the branch, those meetings happened before that. But, when I got here and
21	saw that they hadn't been responded to, I made sure that, as we were going
22	along, my staff started looking at those old comments from 2012 and I said, this
23	is we have to get something done before we're finished because it's not it
24	was not appropriate that we didn't answer him.
25	COMMISSIONER BURNS: Okay, thanks for
26	MR. HISER: And, part of it. I think, that was with our pane

1	process that we took comments like that in. So, it really was until that point that
2	we had resolved the issue. And so, then, we had a response to the comment.
3	COMMISSIONER BURNS: Okay, thanks for that.
4	I think you've recently been before the ACRS on SMR
5	expectancy. When do we expect an ACRS letter on this?
6	MR. WILSON: The letter is completed.
7	COMMISSIONER BURNS: Is it?
8	MR. WILSON: If you would like to see it.
9	COMMISSIONER BURNS: No, that's okay.
10	MR. WILSON: There you go.
11	(LAUGHTER)
12	COMMISSIONER BURNS: So, what did they say and some?
13	I haven't had a chance to
14	MR. WILSON: Well, they said that the documentation, the
15	guidance documents are good and 71003, the inspections will ensure the
16	material condition of the plant along with the audits that will be performed during
17	the review.
18	And, we are ready to receive and evaluate a subsequent
19	license renewal, basically is what the letter says.
20	COMMISSIONER BURNS: Okay.
21	One of the as we've noted, I think, Commission Baran sort of
22	touched on and we talked about a little bit in the first panel, there is some
23	research underway, ongoing, that will inform, you know, long-term, but also
24	short-term, where we go in terms of our evaluation.
25	We've set objectives, which I think is a good thing in terms of
26	how we or how long we anticipate the review would take.

1	What do you see but, I think as the Chairman noted in one of
2	her comments from the last panel is, while we're in a different place, I think, than
3	we were for the when the license renewal rule was first promulgated, because
4	actually we sort of reset the rule in some respects, and we've focused, and
5	think rightly so, on the aging management mechanisms to the extent that we've
5	had this discussion earlier that the living nature of the aging management
7	system within the regulatory context, which I think is a good thing.

But, you know, again, we have some things that may come up as the research concludes. How do you see -- how do you prepare for that with respect to looking at applications that may be underway, that may be in front of us or these initial ones?

And, you know, how do you -- how are you planning for or positioning yourself to work through that in the light of what I would say is an ambitious objective to come through?

MR. WILSON: So, overall, you know, the -- some of the issues where we're doing confirmatory research, the plants will end up sending a plant-specific evaluation that will evaluate their individual aging management program.

We expect them to be a high quality and actually have pre-submittal meetings with the licensee to address something where there's going to be a plant specific evaluation so that we can get a heads up on it.

If, during the process as Commission Baran had asked, if something comes up and we get new information, we will have to ask additional RAIs to address the new information, have the licensee explain how they would deal with that new aging phenomena that we did not have.

And then, forward looking, we would an ISG to send out for all

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1	the future applicants and then make the changes to the SRP and the GALL as
2	necessary once we have written those ISGs.
3	So, we don't anticipate, you know, as we get more and more

2.6

So, we don't anticipate, you know, as we get more and more information, we'll sit back and we'll, you know, if we get -- we'll look at it to see if there's a degradation or a phenomenology that we haven't evaluated and anticipate and starting do that and have industry meetings as stuff comes up with the industry so that they can, based on the frequency as problems arise, so that they can be addressed and put out into the public.

So, that's how right now we would address if something came up unexpectedly.

## COMMISSIONER BURNS: Okay.

And, I, you know, I understand that, you know, the staff has had public meetings to discuss, you know, optimization of the review application or the review and the license renewal application process.

And, you know, some of the issues -- and I've heard this term high quality application from both panels. I hope all applications are high quality for whatever activity that the -- a licensee is seeking either approval from as well as our review are all of high quality.

But, what is it that we need to align? Is there -- are there things we need to align on with respect to the -- what we consider a quality or a high quality application?

MR. WILSON: Well, we can just go by the experience. The industry realized when they submit stuff in, if we don't have questions in that area, that's, you know, what we're looking at.

The SRP and the guidance documents provides a baseline of the stuff that we're looking at. The pre-submittal meetings on stuff that would

take a variance of the GALL or would have something that would be a
plant-specific evaluations, we would say we need this, this, and this to go
through it.

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As long as the licensee and the industry is learning from itself as we go through the process and we're learning, you know, as each submittal comes in, we go through the process. They know exactly what we have to have.

License renewals itself has already laid down a good basis.

They know what they have to give us, to provide us to do a full evaluation of the aging management program.

So, we would expect them to take that as a baseline and then build upon that.

COMMISSIONER BURNS: Okay.

MR. HISER: And, I think one of the main areas it would be completeness of information. And, in some areas, we end up having to ask Requests for Additional Information repetitively, application after application.

And, hopefully, with SLR, some of that information will be built in so we can avoid those RAIs.

COMMISSIONER BURNS: Yes, that's a good point. If we're already able to communicate that. You know, part of it, we're communicating it by asking the -- if we find we that we have to ask the question repeatedly. But, I think there probably is, and obviously, we haven't gotten to the point where we have the applications yet, but they're probably is, if we were sitting here, you know, maybe five, seven years or so from now, and I think just as I think, and my recollection as we did with initial license renewal, sort of stepping back and sort of seeing where, you know, where our areas that, you know, for

Τ	improvement, that we can say, ney locks, if you're really senious about this
2	coming in, you really we need, you know, to be addressing certain areas.
3	You know, the same token, there may be things we put less
4	emphasis on.
5	So, I think that's a good
6	MR. WILSON: The licensees also explained to us that for the
7	subsequent license renewal they're going to require a peer check process to
8	where it wouldn't be just Peach Bottom and Exelon submitting something, North
9	Anna, and Dominion would evaluate Exelon's submittal, then it would be the
LO	same for coming in.
L1	So, that peer check should catch stuff if it's incorrect by the
L2	industry on stuff that we had to have additional information
L3	COMMISSIONER BURNS: And make continuous
L4	MR. WILSON: and make it a higher quality.
L5	COMMISSIONER BURNS: Yes, okay.
L6	Thanks.
L7	CHAIRMAN SVINICKI: Well, thank you all for your
L8	presentations.
L9	I might begin by just making a comment regarding Mr.
20	Lochbaum's observation that he had advanced some comments in 2012 and
21	the staff had not responded for some time.
22	I do want to note that the staff, I think, is exemplary in the
23	amount of public meeting and discussion that it conducts. We do often receive
24	and, of course, benefit our processes benefit in their design and evolution
25	from the input that we receive.
26	We don't always develop a written response document. It is

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1	not always required as part of our processes.
2	What I would hate to see is the staff feel that an expectation is
3	being created that all comment received orally at public meetings required
4	written response within a certain time.
5	I think the inevitable result of that would be that the staff would
6	go out for public dialogue less frequently because there are resource constraints

and, therefore, but I take as a very sound observation that four years is a long time to wait for a reaction.

But, the staff also had the issues in that time under very active

internal deliberation. And, I think perhaps there wasn't a response to give.

So, I respect Mr. Lochbaum deeply and I personally have

benefitted from talking with him and his insights on these issues which he's been following for a long period of time.

But, I don't think that I'm inclined to find fault with the staff in this instance because I think that there's a larger picture here.

And so, I don't expect the staff to react to that. But, I do want to say that, you know, it's the great irony of life sometimes. Your instincts are so pure that you want to go out and have public dialogue but it may be that that creates an expectation of, you know, of a formal written response.

I think we have a limited subset of our processes that actually do require us to do that within a certain period of time. And, that discipline is very beneficial. But, it's not always a part of our public dialogue.

So, I just felt a need to say that. Irrespective of Branch Chiefs transitioning and turning over, I think there was more to that story if the right NRC person had been here.

So, I wanted to advance that on the staff's behalf.

1	Turning to the subject matter that we were talking about directly
2	today, and I think it was Allen, you mentioned that the NRC formed subsequent
3	license renewal expert panels. Those were comprised of folks from NRR and
4	Research and they began in the initial stages of developing the guidance
5	documents for subsequent license renewal.
6	Since we've talked about the aging management, the
7	inspection process, the living nature of that and the feedback loops, how was
8	inspection expertise at NRC represented in those panels? Is NRR bringing it?
9	Is that the representation of it? Is it through the NRR experts?
10	MR. HISER: Actually, though both NRR and Research. We
11	do have staff that change offices. And so, we have a spectrum of experiences
12	and knowledge really in both offices and that's why we form these panels with
13	the two groups.
14	We did have some folks whose expertise is purely inspection.
15	So, they were involved. And, also, from dealing with the issues with plants and
16	with the industry, I think everybody ends up with a certain level of expertise on
17	inspections.
18	CHAIRMAN SVINICKI: Okay, thank you.
19	It just, based on the responses to some of the other questions,
20	it seemed like that specific expertise was necessary for those panels. So, I
21	appreciate your assurance that that was represented in an appropriate form on
22	those.
23	Brian, I was wondering, we've talked a lot about the research

We've had some of the representatives here today. The DOE

that's been done, the ongoing research. There is this integrated and

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coordinated effort.

1	light water reactors sustainability program, EPRI, the industry has talked a little
2	bit about it.
3	Could you describe at a high level what it is that NRC does in
4	its independent confirmatory capacity versus participating in those integrated
5	research programs?
6	And, please don't take from my question that I advocate that we
7	would ever be cost-effective for us to go out and replicate all of the physical
8	research itself.
9	I imagine that we do confirmatory review analyses, our own
10	kind of computational work.
11	But, could you talk at a philosophical level, how do we create
12	that measure of independent assurance on these research results?
13	MR. HISER: Yes. Well, we, you know, we work very closely
14	collaboratively with industry and with EPRI and with DOE.
15	We strive to identify what research is being done and I believe
16	EPRI I believe Sherry spoke about road maps as well as Jason Remer.
17	We develop road maps, we look to see exactly what's needed.
18	But our focus is on what research is being done, how does it enable us to deal
19	with the degradation phenomena or the issues that we're focusing on?
20	But we also look to identify what are the gaps in the research
21	that is not being done?
22	For example, a good example is the irradiated concrete. You
23	know, Sherry spoke about, and Rich Reister spoke about under the LWR's
24	programs, but we had a lot of analytical techniques, their modeling of the
25	concrete and so forth.
26	We felt very strongly that looking at the irradiated damage on

the concrete should be addressed. We communicated that. We identified it.

And then, the staff itself undertook the effort so we initiated those efforts to seek out the concrete from Zorita through our work with Oak Ridge and through the Spanish regulator, obtain that concrete, harvest the concrete, you know, and focused on characterization of that concrete.

And then, also, we discussed that matter with DOE and EPRI and whereby DOE then undertook efforts to help acquire the concrete and the plans to ship that concrete to Oak Ridge, store the concrete. DOE would undertake the economic or the cost associated with that.

At which point we will do the reviews of what is being done analytically. We'll do reviews, re-reviews, if you will, of the modeling and so forth.

But, we will arrive at a point where we'll decide, do we need to go further based on the information we have in hand?

So, there is a hold point, for example, on the irradiated concrete. Do we need to go further and undertake further testing of concrete irradiation, recognizing it's a very involved economic factor there.

So, speaking of it in terms of an appendance, this was under our -- under the NRC staff volition that we encouraged the industry to go in that direction.

CHAIRMAN SVINICKI: So, it sounds like from your answer, I think this is an important reminder, I proposed the question so focused on the back end and when the results are achieved, what do we do with them?

But, I'm widening the aperture based on your response which is that, it is looking maybe at the design of test plans and other things all along the way that allows us to perhaps receive results that we think are going to be

1	the most directly relevant to the safety determinations that we need to make
2	rather than allow that all to happen outside of any input from us as the safety
3	regulator.
4	And, at the end of the day, throw the results back over the
5	transom and say, well, you didn't test this in the areas where I think are most
6	relevant to the determination that I have to make.
7	So, it's really that participation along the way.
8	MR. HISER: Yes, yes.
9	CHAIRMAN SVINICKI: Okay.
10	MR. HISER: It's much of that.
11	CHAIRMAN SVINICKI: Okay, thank you for that.
12	And then, the NEI presenter talked about a draft model SLR
13	new and significant assessment approach for SAMA that they've submitted to
14	the NRC, I imagine for endorsement as a guide.
15	And, the intention articulated by NEI would be to improve the
16	environmental review process.
17	Is there any of the staff here at the table that could talk about
18	our understanding of the purpose of that? Is it submitted for possible NRC
19	endorsement? And, if so, what is the general time frame within which NRC
20	staff would look at and react to that document?
21	Okay. Are we not serious people are being called all the
22	way from the back of the room. I think people are going to start leaving the
23	room.
24	(LAUGHTER)
25	CHAIRMAN SVINICKI: It did not save you, sir, to sit in the
26	back row. You're having to come all the way up here to the microphone.

1	And, if you could please introduce yourself when you before
2	you give this answer that you're the only person in the room who knew it.
3	MR. PARILLO: My name is John Parillo. I'm in Division of
4	Risk Assessment in NRR and I was involved in some of the SAMA reviews.
5	I'm actually here filling in for Jerry Dozier who is more involved
6	with this particular issue. He did the SAMA well the he performed the
7	review for the Limerick which was based on a review of new and significant
8	information.
9	And, we did have a meeting last week or week before
10	discussing this document. I do not know the time frame.
11	CHAIRMAN SVINICKI: Oh, you're really garnering a lot of
12	sympathy, I think, from most of the people in the room.
13	MR. PARILLO: But
14	(LAUGHTER)
15	MR. PARILLO: What I can tell you about it is that the issue will
16	be on providing new and significant information and that discussion is
17	CHAIRMAN SVINICKI: Is this to provide maybe for some, if
18	agreed to, consistency of approach to the
19	MR. PARILLO: Yes.
20	CHAIRMAN SVINICKI: look for new and significant
21	MR. PARILLO: As in how
22	CHAIRMAN SVINICKI: for that
23	MR. PARILLO: in how that
24	CHAIRMAN SVINICKI: a consistent assessment
25	approach?
26	MR PARILLO: Exactly

1	CHAIRMAN SVINICKI: Okay.
2	MR. MCCREE: Chairman, if I might.
3	CHAIRMAN SVINICKI: I'm glad I'm answering my owr
4	questions now.
5	MR. MCCREE: If you would
6	CHAIRMAN SVINICKI: But, go ahead.
7	MR. MCCREE: We'd be happy to provide the
8	CHAIRMAN SVINICKI: Okay.
9	MR. MCCREE: you and the Commission
10	CHAIRMAN SVINICKI: That's fine.
11	MR. MCCREE: information to address
12	CHAIRMAN SVINICKI: And, I think just a note, it does, in al
13	seriousness, and I do thank you. That shouldn't have
14	And, whoever called you up, that was mean.
15	(LAUGHTER)
16	CHAIRMAN SVINICKI: Anyway, but thank you for you
17	answer.
18	I will say that, on a more serious point, you know, we focused
19	a lot on the technical review. But, if there is a broad expectation or if the NRC
20	staff has itself articulated that perhaps an 18-month review schedule is going to
21	be achievable, I think the companion environmental reviews, I would imagine
22	would benefit from the same systematic look and updating of various things.
23	So, although the staff didn't cover this particular submittal in its
24	presentation, it does seem to me to be a complementary type of initiative by the
25	industry to say, hey, what how would NRC react to this as the standardized
26	template for doing these assessments?

1	And, it would be important for the environmental processes to
2	proceed with the same sort of efficient dispatch as everything else.
3	MR. MCCREE: Thank you for the context and we'll ensure
4	that we understand what NEI is proposing or requesting and that a schedule for
5	review doesn't de-compliment the 18-month review schedule which we believe
6	is achievable.
7	CHAIRMAN SVINICKI: All right, thank you.
8	Michele did you want to okay, all right.
9	Just one last thing, and I know we've, on our side of the table,
10	run over, we were chatting in the back that, at some point, when there's more
11	people sitting on this side of the table, we're going to have to be a little more
12	disciplined than we're being these days.
13	But
14	COMMISSIONER BURNS: Just schedule seven minutes.
15	CHAIRMAN SVINICKI: We'll schedule seven minutes, okay.
16	Well, that's we will take that under advisement.
17	(LAUGHTER)
18	CHAIRMAN SVINICKI: Yes, when we've been going to 14
19	and 15 minutes, we're really going to do seven? Okay, well, we'll see how that
20	goes.
21	I wanted to ask about NRR's approach to organizing itself over
22	the longer term for maybe the winding down of initial license renewal
23	applications and moving into subsequent license renewal.
24	The interesting paradigm is that the one is not supposed to be
25	so different from the other. But, the peak resource loading estimated for initial
26	license renewal, there was the CAP 12 policy, it was soft policy, but it meant

1	that we budgeted in general to have 12 in house.
2	I think were some peak years where the industry might have
3	sent in 15 or more.
4	But, in any event, I don't think the numbers are predicted or
5	forecast to get that high in any given year.
6	But, of course, we've had a division of license renewal because
7	you know, to have 12 very active license renewal applications, this is a
8	significant body of work for a group of reviewers.
9	So, over the longer term, as NRR looks at how its organized
10	Michele and then eventual the eventual perhaps folding in of the new reactor
11	work is over the longer term, does license renewal continue to merit a
12	standalone organization or is that not determined at this point?
13	MS. EVANS: So, good question.
14	We have been analyzing this over the last few years. We were
15	working toward a restructured NRR which we intend to move forward to put in
16	place in October.
17	The issue of license renewal and whether there should be a
18	standalone division has been vetted and there are different camps on it. But
19	the happy to say, moving forward, where we're headed is a division of
20	materials and license renewal.
21	And, the you know, there was a strong view that we needed
22	to keep the resources together to project manage to manage the license
23	renewal, subsequent license renewal effort which will be the case with this new
2.4	division.

But, at the same time, there was a need to the current license renewal division does have some technical resources that are placed in that

division and those folks clearly work on license renewal technical issues.

So, as we're moving forward and in working to be more efficient, effective, agile with our staff, there was a camp that felt like the technical resources should all go into the division of engineering, the various technical side of NRR.

So, it turned out that we came up, I think, we've actually -- the staff, in the process of coming up with the new organization, came up with this idea where it would be effective to still have a division of license renewal and predominately the materials engineers, the materials branches support license renewal significantly.

And, operating reactors that we would combine those groups together and they'll be on the technical side of NRR under Brian McDermott on his side. And, we would get the both -- the best of both worlds which would be to still have a division director responsible for license renewal and subsequent license renewal.

And, at the same time, satisfy some of us who, you know, like the structural, the electrical, the various -- those technical disciplines, those folks will be back, you know, they'll be in the division of engineering and will be able to support through like a matrix situation to do the reviews that are necessary.

So, it allows us to, you know, use the staff that we do have to do more work in the technical side than just license renewal. They'll be, you know, a knowledge management issue where the people who have done it can educate the folks who haven't had experience in license renewal and then vice versa and will become more able to handle the workload with, you know, overall, our staff numbers have decreased quite significantly in the last two years. We're down by 20 percent, I think, we're, you know, staff-wise.

1	So, we're trying to structure ourselves to be able to handle the
2	workload in a better way.
3	CHAIRMAN SVINICKI: Okay.
4	MR. WILSON: In addition, the three materials branches were
5	set up and we had a lot of discussion with NRO to make sure when NRR and
6	NRO merge, the materials branches are actually set up by discipline. So, it'll
7	make the merge seamless. And, they're already divided up.
8	Our expectations is right now, we're doing work for NRO in the
9	materials group. So, we can, you know, all the branches will be doing 50, 52
10	and 54, all work simultaneously.
11	CHAIRMAN SVINICKI: Okay, great.
12	Well, I thank everyone for the presentation.
13	Did either of my colleagues have any additional questions?
14	Commissioner Baran?
15	COMMISSIONER BARAN: Can I ask one more? Am I
16	allowed to do that?
17	CHAIRMAN SVINICKI: Sure.
18	COMMISSIONER BARAN: This is a little bit, if we don't have
19	the right folks at the table for this, that's fine, we can talk about it at a different
20	time, but, when you all were talking a little bit about inspections, I was thinking
21	about maintenance which seems like it's going to be increasingly important for
22	plants that are operating past 60 years, if we get to that stage.
23	Is the staff, and I know right now, our inspectors are indirectly
24	assessing maintenance activities through post-maintenance testing and
25	maintenance rule-baseline inspections.
26	Is the staff looking at all or discussing at all whether, you know,

1	for if we have plants operating the 60 to 80-year time frame, whether we would
2	need any adjustments to our maintenance inspections in that period of time?
3	MR. MCCREE: Commissioner, that's a good question.
4	Licensees are still required to comply with 10 CFR 50.65 on the maintenance
5	rule and Appendix B.
6	The aging management programs are required to remain in
7	place under Part 54. And, in addition to the 71002, the prior to transition to the
8	license renewal period and the 71003 which is team inspection afterwards,
9	inspectors will inspect, are inspecting, aging management programs under
10	various baseline inspection procedures, whether it's the maintenance, heat sink,
11	flooding, problem identification and resolution.
12	There are over a dozen procedures where there's a division of
13	inspection and resource support has explicitly included time and samples to
14	inspect the adequate implementation of aging management programs.
15	So, that's a looking forward activity and it is as important today
16	and for those 47 some odd licensees operating in their period of extended
17	operations. It's as relevant today as it will be in the future or even under
18	subsequent license renewal.
19	Having said that, we will factor in operating experience and
20	other intelligence to enhance our procedures as we need to - to ensure that
21	oversight is effective and it's giving us insights that licensees are effectively
22	managing aging, both in active systems as well as passive systems.
23	COMMISSIONER BARAN: Okay, thank you.
24	CHAIRMAN SVINICKI: Thank you.
25	Did you have anything?
26	All right, again, thank you all for your presentations and we are

- 1 adjourned.
- 2 (Whereupon, the above-entitled matter went off the record at
- 3 **12:04 p.m.)**